

December 23, 2004

LICENSEE: Indiana Michigan Power Company
FACILITY: Donald C. Cook Nuclear Plant, Units 1 and 2
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON
NOVEMBER 10, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY
COMMISSION AND INDIANA MICHIGAN POWER COMPANY, PERTAINING
TO THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE
RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the NRC or the staff) and representatives of Indiana Michigan Power Company (I&M) held a telephone conference call on November 10, 2004, to discuss and clarify requests for additional information (RAIs) concerning the Donald C. Cook Nuclear Plant, Units 1 and 2, license renewal application (LRA). The conference call was useful in clarifying the intent of the staff's RAIs.

Enclosure 1 provides a listing of the telephone conference call participants. Enclosure 2 contains the items discussed with the applicant, including a brief description on the status of the items.

The applicant has had an opportunity to comment on this summary.

/RA/

Jonathan G. Rowley, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

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LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE CALL
TO DISCUSS THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION

NOVEMBER 10, 2004

Attendees

Jonathan Rowley
James Strnisha
Thomas Cheng
Rich McNally
Charlie Hoffmeyer
Richard Morante
Robert Kalinowski
Neil Haggerty
Rich Grumbir
Doug Badgero
Allen Cox

Affiliations

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Indiana Michigan Power Company (I&M)
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** I&M Contractor

**REQUESTS FOR ADDITIONAL INFORMATION
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
NOVEMBER 10, 2004**

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Indiana Michigan Power Company (I&M) held a telephone conference call on November 10, 2004, to discuss and clarify requests for additional information (RAIs) concerning the Donald C. Cook Nuclear Plant, Units 1 and 2 (CNP), license renewal application (LRA). The following RAIs were discussed during the telephone conference call.

1. RAI 3.3.2.1.11-1

The applicant submitted the supplemental response to RAI 3.3.2.1.11-1 in a letter to the NRC (AEP:NRC:4034-17) dated October 18, 2004. The staff had an additional question for the applicant following the review of the response.

Additional Information Requested

In the I&M Supplemental Response to RAI 3.3.2.1.11-1, the applicant states, "The remaining systems (CONT, DRAIN, RMS, RWD, and SD) have copper alloy, carbon steel, stainless steel, or glass components that may be pressurized and contain raw water or untreated water. As discussed above, glass exposed to a raw or treated water environment has no aging effect requiring management. I&M will include components containing raw or untreated water subject to aging management review that were included for 10 CFR 54.4(a)(2) in these systems in the Chemistry One-Time Inspection Program." For components in a raw or untreated water environment where an aging effect is likely to occur, the GALL report recommends use of a mitigative program with a One-Time Inspection Program to verify the program effectiveness or the use of periodic inspections. For components in these system, it is not clear to the staff if the applicant is crediting a mitigative program in conjunction with the One-Time Inspection Program or if a one-time inspection alone is credited to manage these aging effects.

Status

The applicant clarified that several A(2) systems contain treated water from the main steam and main feed system and that the water in these system is managed by water chemistry and one-time inspections. One-time inspections will be performed on these systems or will be bounded by other one-time inspections. The applicant maintains that the response to this question has followed previous NRC accepted positions for aging management of A(2) components that are in scope for spray and leakage interaction. The applicant has agreed to provide a written response to further clarify their position regarding the systems that are addressed by this question.

2. RAI 3.4-4

The applicant provided the staff a draft of it's supplemental response to RAI 3.4-4 for the staff's review and comment. The staff had an additional question for the applicant following the review of the response.

Additional Information Requested

The response does not address our comment that the management of loss of preload in bolting is limited to bolts subject to long term exposure to elevated temperatures or significant vibration and not all safety related bolting identified in GALL AMP XI.M18 and NUREG-1339. The applicant is requested to clarify the basis for not managing loss of preload for all safety related bolting and other bolting within scope.

Status

The applicant stated that the response to this question has followed operating experience and previously NRC accepted positions on loss of preload related to bolting and torquing. The staff has requested confirmation of maintenance activities including bolting and torquing. The applicant will provide a written response stating their maintenance activities.

3. RAI 2.4-2(a)

The applicant submitted the supplemental response to RAI 2.4-2(a) in a letter to the NRC (AEP:NRC:4034-11) dated August 11, 2004. The staff had an additional question for the applicant following the review of the response.

Additional Information Requested

The staff indicated that the thermal insulation scoping issue needs to be more specific. The staff requested clarification as to what intended function for license renewal the thermal insulation has and what does the aging management review (AMR) of the insulation include.

Status

The applicant stated that thermal insulation for penetrations does not meet the 10 CFR 54.4 (a)(1), (a)(2), or (a)(3) criteria. The applicant will determine if insulation should be conservatively considered in scope and subject to aging management and will provide this determination in a written response. The staff had no further questions at the time of the call.

4. RAI 2.4-2(b)

The applicant submitted the supplemental response to RAI 2.4-2(b) in a letter to the NRC (AEP:NRC:4034-11) dated August 11, 2004. The staff had an additional question for the applicant following the review of the response.

Additional Information Requested

The staff indicated that the divider barrier seals and the divider barrier hatch seals need to be included in the scope of the Divider Barrier Seal Inspection Program.

Status

The applicant stated that the divider barrier seals and the divider barrier hatch seals will be included in program scope. The applicant will submit a formal response.

5. RAI 2.4-3

The applicant submitted the supplemental response to RAI 2.4-3 in a letter to the NRC (AEP:NRC:4034-11) dated August 11, 2004. The staff had an additional question for the applicant following the review of the response.

Additional Information Requested

The applicant is requested to clarify the details regarding “discharge jets” and trash baskets and collection equipment. Additionally, the staff requested the applicant to identify the components that are relied on for de-icing to ensure an adequate supply of cooling water for safe shutdown, and to verify these components are included in the license renewal scope.

Status

The applicant stated the request is clear. The applicant will submit a formal response.

6. RAI 2.4-5

The applicant submitted the supplemental response to RAI 2.4-5 in a letter to the NRC (AEP:NRC:4034-11) dated August 11, 2004. The staff had an additional question for the applicant following the review of the response.

Additional Information Requested

The staff requested that the applicant clarify which aging management program (AMP) is credited to ensure structural performance of cable feed-through assemblies.

Status

The staff position is that the assemblies perform a pressure boundary intended function, thus requiring an AMP. The staff believes this is consistent with 10 CFR 50, Appendix J. The applicant will reevaluate and respond in a formal letter.

Donald C. Cook Nuclear Plant, Units 1 and 2

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