



Entergy Nuclear Northeast  
Entergy Nuclear Operations, Inc.  
440 Hamilton Avenue  
White Plains, NY 10601  
Tel 914 272 3200  
Fax 914 272 3205

Michael R. Kansler  
President

December 17, 2004  
NL-04-161

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Subject: Indian Point Nuclear Generating Unit No. 2  
Docket No. 50-247  
Indian Point Nuclear Generating Unit No. 3  
Docket No. 50-286  
**Indian Point 2 and Indian Point 3 Annual 10 CFR50.46 Report  
for Year 2003, and Indian Point 2 10 CFR 50.46 30-Day Report**

- References:
1. Westinghouse letter (IPP-04-126), S. Ira to C. Schwarz, "Revised PCT Rackups for 10 CFR 50.46 Annual Notification and Reporting for 2003", dated October 4, 2004.
  2. Entergy letter (NL-03-111), M. Kansler to NRC, "Annual 10 CFR 50.46 Report for Year 2002 - Emergency Core Cooling System Evaluation Changes", dated, June 24, 2003.
  3. Entergy letter (NL-04-005), F. Dacimo to NRC, "Proposed Changes to Technical Specifications: Stretch Power Uprate Increase of Licensed Thermal Power (3.26%)", dated January 29, 2004.
  4. Entergy letter (NL-04-081), F. Dacimo to NRC, "Proposed Schedule for Reanalysis of Large Break Loss Coolant Accident", dated July 2, 2004.

Dear Sir:

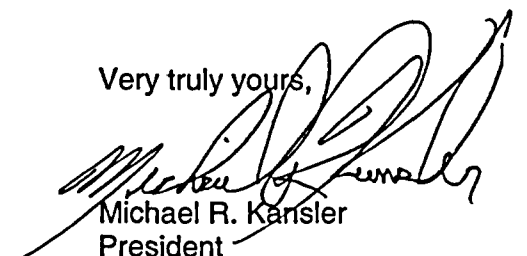
Pursuant to the 10 CFR 50.46 Annual Notification requirements, Entergy Nuclear Operations, Inc. (ENO) has reviewed the 2003 Small and Large Break Loss of Coolant Accident (LOCA) peak clad temperature (PCT) changes described in Reference 1, for Indian Point Unit No. 2 and Indian Point Unit No. 3. The Indian Point Unit No. 2 and Indian Point Unit No. 3 PCTs remain below the maximum fuel cladding temperature of 2200°F. Reference 2 provided the Indian Point Unit No. 2 and the Indian Point Unit No. 3 Annual 10 CFR 50.46 Report for 2002.

Reference 3 submitted to the NRC the Indian Point Unit No. 2 Stretch Power Uprate proposed Technical Specification changes and supporting analyses. The Large Break LOCA (LBLOCA) analysis documented a peak clad temperature change of greater than 50°F per the definition of significant change in 10 CFR 50.46. Therefore, this letter, also, constitutes ENO's 30-day report pursuant to 10 CFR 50.46(a)(3)(ii). Subsequent to discussions with the NRC staff, ENO provided a schedule for reanalysis of the Indian Point Unit No. 2 LBLOCA analysis. The aforementioned schedule was provided to the NRC in Reference 4.

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There are no new commitments made in this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,



Michael R. Kansler  
President  
Entergy Nuclear Operations, Inc.

cc: Mr. Patrick D. Milano, Senior Project Manager  
Project Directorate I,  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Mail Stop O-8-C2  
Washington, DC 20555

Mr. Samuel J. Collins  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Senior Resident Inspector's Office  
Indian Point Unit 2  
U.S. Nuclear Regulatory Commission  
P.O. Box 59  
Buchanan, NY 10511-0059

Resident Inspector's Office  
Indian Point Unit 3  
U.S. Nuclear Regulatory Commission  
P.O. Box 337  
Buchanan, NY 10511-0337

Mr. Peter R. Smith  
President  
NYSERDA  
17 Columbia Circle  
Albany, NY 12203

Mr. Paul Eddy  
New York State Dept. of Public Service  
3 Empire Plaza  
Albany, NY 12223