December 21, 2004

LICENSEE: Nuclear Management Company, LLC

FACILITY: Point Beach Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON OCTOBER 28, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference on October 28, 2004, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application. The conference call was useful in clarifying the intent of the staff's D-RAIs.

Enclosure 1 provides a listing of the meeting participants. Enclosure 2 contains a listing of the D-RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/
Michael J. Morgan, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures: As stated

cc w/encls: See next page

December 16, 2004

LICENSEE: Nuclear Management Company, LLC

FACILITY: Point Beach Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON OCTOBER 28, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION

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Docket Nos. 50-266 and 50-301

Enclosures: As stated

cc w/encls: See next page

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DISTRIBUTION: Note to Licensee: Nuclear Management Co., LLC , Re: Point Beach Nuclear Plant, Units 1 and 2, Dated: December 21, 2004

Adams accession no.: **ML043570259**

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LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE
TO DISCUSS THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
OCTOBER 28, 2004

Participants	Affiliations
B.Fromm	Nuclear Management Company, LLC
J.Knorr	Nuclear Management Company, LLC
T.Mielke	Nuclear Management Company, LLC
M.Hartzman	Nuclear Regulatory Commission
M.Heath	Nuclear Regulatory Commission
M.Morgan	Nuclear Regulatory Commission

DRAFT REQUESTS FOR ADDITIONAL INFORMATION (D-RAI)
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
October 28, 2004

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on October 28, 2004, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application (LRA). The following D-RAIs were discussed during the telephone conference call.

4.3 Metal Fatigue

D-RAI-4.3.1

Provide confirmation that the limiting locations of the PBNS reactor vessels evaluated for extended operation correspond to the structures and/or components listed in Table IV.A2 of NUREG-1801, Volume 2, for PWR reactor vessels structures and/or components, where cumulative fatigue damage/fatigue is the aging effect/mechanism, and which require further evaluation as TLAAs for the period of extended operation. Alternatively, provide the location in the LRA where this information is shown.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.2.1

Provide confirmation that the limiting locations of the PBNS reactor vessel internals evaluated for extended operation correspond to the structures and/or components listed in Table IV.B2 of NUREG-1801, Volume 2, for PWR reactor vessel internals structures and/or components, where cumulative fatigue damage/fatigue is the aging effect/mechanism, and which require further evaluation as TLAAs for the period of extended operation. Alternatively, provide the location in the LRA where this information is shown.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.2.2

Provide a summary of 60-year primary-plus-secondary stress intensities and cumulative fatigue usage factors (similar to revised Tables 4.3-1 and 4.3-2 in Appendix A of the LRA for components of the reactor vessel) for the key reactor internal components listed on page 4-41 of the LRA.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.3

Provide a comparison of the CLB set of transient conditions and design cycles, and the revised set of full power uprate transient conditions and design cycles, that were used in the CRDM fatigue TLAAs to show conformance with the CLB fatigue limits to the end of the period of extended operation.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.4.1

Provide confirmation that the limiting locations of the PBNS steam generators evaluated for extended operation correspond to the structures and/or components listed in Table IV.D1 of NUREG-1801, Volume 2, for PWR reactor vessels structures and/or components, where cumulative fatigue damage/fatigue is the aging effect/mechanism, and which require further evaluation as TLAAs for the period of extended operation. Alternatively, state the location in the LRA where this information has been provided.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.4.2

Provide a comparison of the CLB set of transient conditions and design cycles, and the revised set of Steam Generator Replacement and Full Power Uprate transient conditions and design cycles, that were used in the Units 1 and 2 steam generator fatigue TLAAs to show conformance with the CLB fatigue CUF limit to the end of the period of extended operation. Alternatively, provide clarification stating that the applicable transient conditions and design cycles are those stated in Table 4.1-8 of Appendix A to the LRA.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.4.3

List the key Units 1 and 2 steam generator components, and provide for each a summary of 60-year primary-plus-secondary stress intensities and cumulative fatigue usage factors (similar to revised Tables 4.3-1 and 4.3-2 in Appendix A of the LRA for components of the reactor vessel) for these components.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.5.1

Provide confirmation that the limiting fatigue locations of the PBNS pressurizers evaluated for extended operation correspond to the pressurizer structures and/or components listed in Table IV.C2.5 of NUREG-1801, Volume 2, for PWR reactor vessels structures and/or components, where cumulative fatigue damage/fatigue is the aging effect/mechanism, and which require further evaluation as TLAAs for the period of extended operation. Alternatively, state the location in the LRA where this information has been provided.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.5.2

Provide a comparison of the CLB set of transient conditions and design cycles, and the revised set of Steam Generator Replacement and Full Power Uprate transient conditions and design cycles, that were used in the Units 1 and 2 pressurizers fatigue TLAAs to show conformance with the CLB fatigue limit to the end of the period of extended operation.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.5.3

Provide clarification that the “plant-specific insurge/outsurge” fatigue analyses are based on the combination of the insurge/outsurge transient condition and the transients listed in the revised set of Steam Generator Replacement and Full Power Uprate transient conditions.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.5.4

Provide a description of the “Modified Operating Procedures” (page 4-45) that were used to minimize or eliminate in-surge/out-surge cycling.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.5.5

List the key Units 1 and 2 pressurizer components, and provide for each a summary of 60-year primary-plus-secondary stress intensities and cumulative fatigue usage factors (similar to revised Tables 4.3-1 and 4.3-2 in Appendix A of the LRA for components of the reactor vessel) for these components.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAN-4.3.7

Provide a comparison of the CLB set of transient conditions and design cycles, and the revised set of Steam Generator Replacement and Full Power Uprate transient conditions and design cycles, that were used in the Units 1 and 2 pressurizer surge line fatigue TLAAs to show conformance with the CLB fatigue limit to the end of the period of extended operation.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI-4.3.8

This section states that: "In view of the lack of margin with the Unit 1 piping system analysis results for end of life extension (EOLE), additional analysis investigations were pursued. The original 88-08 analysis incorporated simplified analysis techniques and assumptions. It was not clear that the analysis was in fact conservative. The 88-08 analyses were re-performed using the original temperature monitoring data, and refined analysis techniques and assumptions." Provide a detailed description and basis of the "refined analyses techniques and assumptions" that were used in the 88-08 re-evaluation to reduce the 60-year CUF of 0.99 for the Unit 1 piping system to a 60-year CUF of 0.277.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI- 4.3.10.1

For the USAS B31.1 locations, provide a description of the PBNP-specific simplified ASME Section III fatigue analyses that were used to calculate environmentally based cumulative usage factors.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

D-RAI- 4.3.10.2

The Pressurizer CUFs are determined based on EPRI MRP-47 methodology. The staff has not endorsed MRP-47. Provide the environmentally assisted CUFs for the pressurizer locations, based on the staff-accepted methodology as stated in Sections 4.3.2.2 and 4.3.3.2 of NUREG-1800.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.