### January 12, 2005

### ORGANIZATION: ATOMIC ENERGY OF CANADA LIMITED (AECL) TECHNOLOGIES INC

# SUBJECT: SUMMARY OF MEETING HELD ON DECEMBER 2, 2004, TO DISCUSS ACR-700 REACTOR PHYSICS CODES

The Nuclear Regulatory Commission (NRC) hosted a public meeting with Atomic Energy of Canada Limited (AECL) Technologies, Inc. on December 2, 2004, at the U.S. Nuclear Regulatory Commission (NRC) Headquarters in Rockville, MD. The purpose of this meeting was to discuss the main elements of the reactor physics program for the Advanced CANDU Reactor (ACR-700) design. For a list of meeting attendees refer to Attachment 1.

AECL Technologies Inc. provided the NRC staff with the following presentations:

- C ACR Physics Program Overview
- C ACR Physics Codes and Models Assessment, Development and Verification
- C ACR ZED-2 Physics Measurements
- C ACR Physics Toolset Validation

The ACR-700 physics program was updated to support design certification requirements and AECL plans to continue to improve their toolset beyond the design certification application submittal through their nuclear platform research and development (R&D) program. AECL stated they plan to provide a report in January 2005 describing their code development and experimental program. In February 2005, they plan to provide, for the staff's review, a checkerboard voiding assessment report that includes a coupled multi-channel thermal hydraulics and reactor physics large break loss-of-coolant accident (LBLOCA) simulation. In addition, AECL plans to provide the details of the Zero Energy Deuterium-2 (ZED-2) facility requested by the staff and a report addressing the suitability of ZED-2 for ACR-700 experiments.

The experimental components of the ACR-700 physics program include: checkerboard voiding measurements, reactivity coefficients including coolant void reactivity (CVR), fuel temperature, and partial void reactivity, fine-structure measurements, reactivity device measurements, and kinetics measurements.

AECL's validation database will include data from other critical facilities; such as, D/sub 2/O Critical Assembly (DCA) facility in Japan, the Deuterium Moderated Pile Low Energy Reactor (DIMPLE) facility in the United Kingdom, ORGEL Critical Experiment (ECO) facility in Italy, and the Savannah River facility in the United States.

The meeting concluded with a discussion on the key review issues identified in the ACR-700 pre-application safety assessment report (PASAR). AECL stated that they intend to address as many issues as possible before submitting the design certification application. AECL suggested quarterly progress update meetings throughout phase three of the pre-application review to update the staff on the progress of the ACR-700 reactor physics program.

Additional details on the material covered in this meeting may be accessed through the ADAMS system under Accession No. ML043410254. If you do not have access to ADAMS or if there are problems in accessing the handouts located in ADAMS, contact the NRC Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov.

Members of the public were in attendance but did not make public comments.

/**RA**/

Belkys Sosa, Project Manager New Reactors Section New, Research and Test Reactors Program Division of Regulatory Improvement Programs Office of Nuclear Reactor Regulation

Project No. 722

Enclosures: As stated

cc w/encls: See next page

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## ACR-700 Reactor Physics Codes

## December 2, 2004 O-10B4 8:30AM - 4:45 PM

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Attachment 1

#### ACR-700

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