



Serial: RNP-RA/04-0150

DEC 13 2004

United States Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261/LICENSE NO. DPR-23

ANNUAL REPORT OF CHANGES TO OR ERRORS DISCOVERED  
IN AN ACCEPTABLE LOSS-OF-COOLANT ACCIDENT EVALUATION  
MODEL APPLICATION FOR THE EMERGENCY CORE COOLING SYSTEM

Ladies and Gentlemen:

In accordance with the provisions of the Code of Federal Regulations, Title 10, Part 50.46, (10 CFR 50.46) Progress Energy Carolinas, Inc., also known as Carolina Power and Light Company, is submitting the attached report of non-significant changes to and errors discovered in an acceptable Loss-of-Coolant Accident (LOCA) evaluation model (EM) for the Emergency Core Cooling System at the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The applicable LOCA EMs are referenced in the HBRSEP, Unit No. 2, Core Operating Limits Report. Non-significant changes to or errors discovered in EMs and EM applications were previously reported to the Nuclear Regulatory Commission by letter dated December 12, 2003. This submittal satisfies the 10 CFR 50.46 requirement for annual reporting of LOCA EM changes for HBRSEP, Unit No. 2.

The non-significant changes and error corrections in the Framatome-ANP (FANP) SEM/PWR-98 Large Break LOCA (LBLOCA) EM and the FANP EXEM PWR Small Break LOCA (SBLOCA) EM since the last annual report are provided in Attachment I. The effects of these non-significant changes and error corrections on HBRSEP, Unit No. 2, peak cladding temperature (PCT) estimates are also summarized in Attachment I.

The latest PCT estimates for the LBLOCA and SBLOCA are included in Attachment II.

If you have any questions concerning this matter, please contact me at (843) 857-1253.

Sincerely,

A handwritten signature in black ink that reads 'C. T. Baucom'.

C. T. Baucom  
Supervisor – Licensing/Regulatory Programs

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United States Nuclear Regulatory Commission

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Attachments:

- I. Report of Changes/Errors in Loss-of-Coolant Accident Evaluation Models for the Emergency Core Cooling System
- II. Peak Cladding Temperature Estimates

c: Dr. W. D. Travers, NRC, Region II  
Mr. C. P. Patel, NRC, NRR  
NRC Resident Inspector, HBRSEP

**H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2**

**REPORT OF CHANGES/ERRORS IN LOSS-OF-COOLANT ACCIDENT  
EVALUATION MODELS FOR THE EMERGENCY CORE COOLING SYSTEM**

This report provides an estimate of the effect on peak cladding temperature (PCT) of non-significant changes and error corrections in the Loss-of-Coolant Accident (LOCA) evaluation models (EMs) and EM applications for the Emergency Core Cooling System (ECCS) at the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2, covering the period of December 13, 2003, through November 29, 2004.

**Large Break Loss-of-Coolant Accident (LBLOCA) Evaluation Model**

<b>CHANGED CONDITION</b>	<b>PCT IMPACT (°F)</b>
None	0
<b>Cumulative Impact</b>	<b>0</b>

**Small Break Loss-of-Coolant Accident (SBLOCA) Evaluation Model**

<b>CHANGED CONDITION</b>	<b>PCT IMPACT (°F)</b>
None	0
<b>Cumulative Impact</b>	<b>0</b>

**H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2**

**PEAK CLADDING TEMPERATURE ESTIMATES**

The current peak cladding temperature (PCT) estimates associated with Loss-of-Coolant Accident (LOCA) Emergency Core Cooling System (ECCS) evaluation models are listed below. These estimates include the cumulative effects of significant and non-significant error corrections and evaluation model changes through November 29, 2004.

<b><u>Event</u></b>	<b><u>PCT (°F)</u></b>
Large Break LOCA, ECCS Injection Mode	2018
Small Break LOCA, ECCS Injection Mode	1679