

December 15, 2004

MEMORANDUM TO: Darrell J. Roberts, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Victor Nerses, Senior Project Manager, Section 2 */RA/*
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: SEABROOK STATION, UNIT NO. 1, FACSIMILE TRANSMISSION,
DRAFT REQUEST FOR ADDITIONAL INFORMATION (TAC NO.
MC2364)

The attached draft Request for Additional Information (RAI) was transmitted by facsimile on December 15, 2004, to Mr. M. O'Keefe of FPL Energy Seabrook, LLC (FPL Energy). This draft RAI was transmitted to facilitate the technical review being conducted by Nuclear Regulatory Commission (NRC) staff and to support a conference call with FPL Energy in order to clarify certain items. The draft RAI is related to the licensee's March 17, 2004, application for stretch power uprate. Review of the RAI would allow the licensee to determine and agree upon a schedule to respond to the RAI. This memorandum and the attachment do not convey or represent an NRC staff position regarding the licensee's prior response.

Docket No. 50-443

Enclosure: Draft Request for Additional Information

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DRAFT REQUEST FOR ADDITIONAL INFORMATION

RELATED TO STRETCH POWER UPRATE

FPL ENERGY SEABROOK, LLC

SEABROOK STATION

DOCKET NO. 50-443

Section 5.2.2.2 (Section 5.2, Page 5) of the Seabrook Updated Final Safety Analysis Report states that an evaluation and analyses of the capability and performance of these valves is presented in Reference 2 (see Section 5.2). Reference 2 identifies WCAP-7769, Revision 1, "Overpressure Protection for Westinghouse Pressurized Water Reactors," June 1972 as the licensing basis report for these specific pressure relieving safety valves. Reference 2 also identifies a letter NS-CE-622, April 16, 1975, from Westinghouse to NRC (then AEC) which contains additional information related to WCAP-7769, Revision 1. The staff has the 1972 report, but not the 1975 letter. Based on review of the Seabrook proposed power uprate and comparison of parameters with those of the largest plant analyzed in WCAP-7769, it is not clear that WCAP-7769 continues to apply to Seabrook operating at the uprated power because the uprated power for Seabrook appears to be higher than the power of the uprated plant analyzed in the report. Show that WCAP-7769 continues to apply to Seabrook operating at the proposed uprated power. One way to demonstrate that the report applies might be to show that for Seabrook operating at the uprated power, the ratio of the safety valve flow to the peak surge rate for Seabrook operating at the uprated power is greater than 1.0 (see WCAP-7769, Table 2-2, Figures 2-2, Table 2-3, and related discussions).

Enclosure