

### EXAM QUESTION HISTORY

Question #                      RO \_\_\_\_\_                      SRO   1    
                                          TIER   1                                        Group   1    
B000.1057                      KA   000017G2.1.32                        Importance   3.8  

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

<u>  X  </u>	New
_____	Modified (Attach original and Modified Questions)
_____	Original Bank _____ # _____
_____	Bank Originating Bank _____ # _____

10CFR55 Content                      55.41 \_\_\_\_\_                      55.43   5  

**Learning Objective**

RES02C 2.01 Given a set of plant and equipment conditions evaluate the conditions to determine the applicable procedure, and from the procedure determine the appropriate EXPECTED ACTIONS or RESPONSE NOT OBTAINED instructions to implement.(ES-0.2)

Cognitive Level                      Memory or Fundamental Knowledge                      \_\_\_\_\_  
                                          Comprehension or Analysis                                        X  

Technical Reference   Figure 3.0, ES-0.2  

Level of Difficulty (from attachment 3) \_\_\_\_\_

References required on Exam (Attach copy to this attachment)

  Fig-3.0, Natural Circulation C/D with Shroud Fans.  

AnswerAnalysis:   D is correct since the ΔT between the Hot legs and the Przr exceeds its 200°F limit. No restrictions are imposed on Aux. Spray because the ΔT is < 320°F.

Distractor Analysis: A is incorrect since the RCS pressure needs to be reduced to  $< 1400$  per

Figure 3.0 and no restrictions are placed on Aux. Spray (used  $T_H$  to calculate RCS pressure).

B is incorrect since no restrictions are placed on Aux. Spray in these conditions.

C is incorrect due to the wrong RCS pressure.

Question 1 B000.1057 (1 point(s))

The plant trip due to loss of offsite power. The RCS is currently being cooled down per ES-0.2, Natural Circulation Cooldown. The following conditions exist:

- RCS Pressure - 1850 psig
- RCS Hot Leg Temps - A - 410°F B - 405°F
- RCS Cold Leg Temps - A - 390°F B - 385°F
- Przr Water Temp - 625°F
- Przr Steam Temp - 626°F
- Letdown In Service
- Charging from the Regen Hx Temp 385°F
- SI Blocked
- Both Control Rod Shroud Fans are running

Which of the following states the required actions:

- a. Depressurize the RCS to less than 1540 psig using only the PORVs due to high  $\Delta T$  between the Charging Line and Przr Steam space.
- b. Depressurize the RCS to less than 1400 psig using only the PORVs due to high  $\Delta T$  between the Charging Line and Przr Steam space.
- c. Depressurize the RCS to less than 1540 psig using auxiliary spray to cooldown the pressurizer.
- d. Depressurize the RCS to less than 1400 psig using auxiliary spray to cooldown the pressurizer.

Answer 1

- d. Depressurize the RCS to less than 1400 psig using auxiliary spray to cooldown the pressurizer.

## EXAM QUESTION HISTORY

Question # B000.1058      RO                 SRO   2    
                                          TIER   1        Group   1    
                                          KA   000022AA2.04        Importance   3.8  

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

<u>  X  </u>	New
<u>          </u>	Modified (Attach original and Modified Questions)
<u>          </u>	Original Bank <u>                          </u> # <u>                          </u>
<u>          </u>	Bank Originating Bank <u>                          </u> # <u>                          </u>

10CFR55 Content      55.41                 55.43   5  

Learning Objective: RAP31C 1.03

State the reason/basis for the CAUTIONS, NOTES and/or Major Action Categories in AP-CVCS.3, Loss of All Charging Flow.

Cognitive Level      Memory or Fundamental Knowledge                  
                                          Comprehension or Analysis        X  

Technical Reference   AP-CVCS.3  

Level of Difficulty (from attachment 3)                           

References required on Exam (Attach copy to this attachment)

  None  

---

Answer Analysis:   B is correct since the guidance of AP-CVCS.3 requires this action.  

---



Question 2 B000.1058

(1 point(s))

While at 100% power a charging line leak resulted in loss of all charging flow. It will be approximately 24 hours before the line can be repaired and charging restored. Which of the following describes the required operator actions under these conditions. The operators have already isolated Letdown.

- a. Shutdown the Rx per the Normal Operating Procedures (10%/hour) once shutdown depressurize the RCS to ~1400 psig and establish SI flow to control inventory. RCS inventory will remain in the pressurizer due to the low outflow rate.
- b. Perform a Rapid Rx Shutdown using Abnormal Procedures (5%/min) once shutdown depressurize the RCS to ~ 1400 psig and establish SI to control inventory. RCS inventory will be maintained due to the short time taken to shutdown.
- c. No action is required since the pressurizer out flow rate (seal leakoff) is so low that more than the required 24 hours for the charging line repair are available prior to of Przr Level decreasing to < 13%.
- d. Monitor Przr Level when Przr Level decreases to less than 5% the Trip the Reactor and go to E-0, SI will automatically start as pressure decrease and SI flow will maintain RCS Inventory.

Answer 2

- b. Perform a Rapid Rx Shutdown using Abnormal Procedures (5%/min) once shutdown depressurize the RCS to ~ 1400 psig and establish SI to control inventory. RCS inventory will be maintained due to the short time taken to shutdown.





Question 3 B008.0005 (1 point(s))

During an ordered 1% per min power reduction, the following annunciators and conditions exist:

- AA-18 RCP Vibration Alert
- A-32 RCP B oil level  $\pm 1.25$
- PPCS display "GD-RCP's" indicates "B" RCP upper bearing at 233 degrees F

Which of the following actions are required?

- a. Check CCW pump, CCW surge tank and surge tank vent.
- b. Trip the Rx, trip both RCP's and go to E-0.
- c. Trip both RCP's, verify RCS isolated and go to E-0.
- d. Trip the Rx, trip "B" RCP and go to E-0.

Answer 3

- d. Trip the Rx, trip "B" RCP and go to E-0.





Question 4 C000.0266 (1 point(s))

ER-PRZR.1, Restoration of PRZR HTRS during blackout, provides for establishing a minimum amount of PRZR heaters.

Which one of the following statements describes the basis or reason for establishing heaters following a blackout?

- a. Allows for keeping the PRZR saturated for more uniform pressure control.
- b. Allows subcooled natural circulation to be maintained.
- c. Allows for collapse of steam bubble in the vessel head.
- d. Allows for maintaining RCS pressure above SI setpoint.

Answer 4

- b. Allows subcooled natural circulation to be maintained.

TC #96-107





Question 5 B000.1059

(1 point(s))

Following a loss of all AC power the operators are performing ECA-0.0, Loss of all AC, when Energy Operations notifies the Control Room that Circuit 751 has been restored. The following conditions exist:

- SI Actuated and not reset
- No Faults on any 4160V or 480V Bus
- All Buses are deenergized

Which of the following states the required actions the operators will take to restore power to the 480V Safeguards Buses.

- a. Restore power to the 4160V Buses 12A/12B per ER-ELEC.1, Restoration of Offsite Power, then Reset SI to allow closing of the 480V Safeguard Buses Normal Feed Breakers.
- b. Restore power to the 4160V Buses 12A/12B per ECA-0.0, Loss of All AC Power, then Reset SI to allow closing of the 480V Safeguard Buses Normal Feed Breakers.
- c. Restore power to 4160V Buses 12A/12B per ER-ELEC.1. The Safeguard Bus Feed Normal Breakers can be closed without resetting SI.
- d. Restore power to 4160V Buses 12A/B per ECA-0.0, Loss of all AC. The Safeguard Bus Feeder Normal Breakers can be closed without resetting SI.

Answer 5

- a. Restore power to the 4160V Buses 12A/12B per ER-ELEC.1, Restoration of Offsite Power, then Reset SI to allow closing of the 480V Safeguard Buses Normal Feed Breakers.





Question 6 B000.1060 (1 point(s))

During 100% power operation, the 1B AC Instrument Bus develops a fault and is deenergized. Assuming that the Bus is not repaired, which of the following states the latest time the NRC can be notified and still meet the notification requirements.

- a. One hour after the event.
- b. Four hours after the event.
- c. Six hours after the event.
- d. Twenty-four hours after the event.

Answer 6

- c. Six hours after the event.





Question 7 B035.0004 (1 point(s))

Immediately following the completion of the immediate actions of E-0, Reactor Trip or Safety Injection, the following conditions exist:

- SI Not Actuated
- RCS pressure - 2180 psig
- Core Exit TCs - 542 degrees F
- S/G pressures - (A,B) - 1000 psig, 900 psig
- S/G N.R. levels (A,B) - 0%, 3%
- Containment pressure - 0.3 psig
- Pressurizer level - 3%
- Total feed flow to S/G A - 50 gpm
- Total feed flow to S/G B - 100 gpm

Assuming these conditions exist and cannot be changed as you progress through the procedure, subsequent action should be performed under which of the following procedures?

- a. ES-0.1, Reactor Trip Response
- b. E-2, Faulted Steam Generator Isolation
- c. FR-H.1, Response to Loss of Secondary Heat Sink
- d. FR-I.2, Response to Low Pressurizer level

Answer 7

- c. FR-H.1, Response to Loss of Secondary Heat Sink





6000.1061

.069.AA2.02

3/10/2003

Indian Point 2 (Unit)

Exam Level S



Print Record

New Search

Exit

Question

The Reactor Coolant System is in a reduced inventory condition and preparations are being made to detention the RV Head for eventual removal. Additional plant status information is as follows:

equipment hatch

- o RCS boron concentration is <sup>2320</sup>2000 ppm *The removed from service*
- o ~~Zone II of the Weld Channel Penetration Pressurization System (WCPS)~~ *was tagged out 48 hours ago*
- o Both doors of the 95 containment airlock are open to facilitate repairs of the ~~WCPS~~ airlock seals
- o One door of the 80 personnel airlock is properly closed
- o Containment leakage was previously verified to be less than 0.2% of the containment free volume per day
- o ~~Containment Purge is in service~~

Given the above plant conditions, which ONE (1) of the following describes a requirement that must be met prior to proceeding with the RV Head detentioning?

Answer:

**A** Repair and properly close at least ONE door in the 95 personnel airlock.

Distracter 1

**B** ~~Verify the Isolation Valve Seal Water System (IVSWS) is OPERABLE. Restore the Containment Penetration Pressurization system to operable status.~~

Distracter 2

**C** ~~Verify the automatic containment purge and procure relief isolation valves are set to limit travel to less than or equal to 80 degrees travel.~~ *is removed from service*

Distracter 3

**D** ~~Close the other door of the 80 personnel airlock.~~ *cont equipment hatch Personnel Airlock*

Distracter Analysis:

Answer:

- A. Correct.
- B. Incorrect. IVSW for Hot Shutdown and above
- C. Incorrect. Purge may be in operation in CSD
- D. Incorrect. Only 1 door required

Distracter 1:

- A. Correct.
- B. Incorrect. IVSW for Hot Shutdown and above
- C. Incorrect. Purge may be in operation in CSD
- D. Incorrect. Only 1 door required

Distracter 2:

- A. Correct.
- B. Incorrect. IVSW for Hot Shutdown and above
- C. Incorrect. Purge may be in operation in CSD
- D. Incorrect. Only 1 door required

Distracter 3:

- A. Correct.
- B. Incorrect. IVSW for Hot Shutdown and above
- C. Incorrect. Purge may be in operation in CSD
- D. Incorrect. Only 1 door required

ANS A.

Question 8 B000.1061 (1 point(s))

The Reactor Coolant System is in a reduced inventory condition and preparations are being made to detention the RV Head for eventual removal. Additional plant status information is as follows:

- RCS boron concentration is 2320 ppm.
- The Penetration Pressurization System was removed from service 48 hours ago
- Both doors of the Equipment Hatch Airlock are open to facilitate repairs of the airlock seals
- One door of the Personnel Airlock is properly closed
- Containment leakage was previously verified to be less than 0.2% of the containment free volume per day
- Containment purge is in service

Given the above plant conditions, which ONE (1) of the following describes a requirement that must be met prior to proceeding with the RV Head detentioning?

- a. Repair and properly close at least ONE door in the Equipment Hatch.
- b. Restore the Containment Penetration Pressurization System to operable status.
- c. Verify the automatic containment purge is removed from service.
- d. Close the other door of the Personnel Airlock.

Answer 8

- a. Repair and properly close at least ONE door in the Equipment Hatch.





Question 9 B000.1062 (1 point(s))

During a High Reactor Coolant Activity event, which of the following is the criteria used to determine if the standby mixed bed demineralizer should be placed in service?

- a. Chloride levels greater than 0.15.
- b. Decontamination factor less than 10.
- c. Dose equivalent I-131 greater than 1 microcurie/gram.
- d. Gross radioactivity greater than 100/E BAR.

Answer 9

- b. Decontamination factor less than 10.





Question 10 B000.1063

(1 point(s))

The Reactor has tripped due to a loss of offsite power. SI has actuated. The crew is performing actions in E-0, "Reactor Trip or Safety Injection". Given the following conditions:

RCS pressure 1700 psig and trending up  
SG pressure "A" = 1015 psig stable "B" = 700 psig and trending down  
CET's 700°F and trending up  
SG Narrow Range level off scale Low  
AFW flow approximately 75 gpm to each SG  
PRZR level 15% and trending down  
CNMT pressure 5 psig and trending up  
Power is 2% in the PR and IR SUR is slightly negative  
RVLIS level 45%

Which ONE of the following describes the procedure transition upon exit from E-0?

- a. E-2, Faulted Steam Generator Isolation
- b. FR-H.1, Response to Loss of Secondary Heat Sink
- c. FR-S.1, Response to Reactor Restart/ATWS
- d. FR-C.1, Response to Inadequate Core Cooling

Answer 10

- d. FR-C.1, Response to Inadequate Core Cooling





Question 11 C000.0043

(1 point(s))

FR-P.1, Response to Pressurized Thermal Shock, Step 17, has a caution which states, "The RCS should not be depressurized to less than SI accumulator pressure until the SI accumulator is isolated." Which one of the following is the correct basis for this caution?

- a. Depressurization to less than accumulator pressure prior to accumulator isolation may cause water solid PRZR which will result in a loss of PRZR pressure control.
- b. Depressurization to less than accumulator pressure prior to accumulator isolation will allow nitrogen into the RCS which may cause gas binding in the SG U-tubes.
- c. Depressurization to less than accumulator pressure prior to accumulator isolation will delay depressurization since accumulator in flow to RCS will tend to stabilize RCS pressure.
- d. Depressurization to less than accumulator pressure prior to accumulator isolation will cause cold water addition to RCS which will result in rapid RCS depressurization and loss of subcooling from thermal shrink.

Answer 11

- c. Depressurization to less than accumulator pressure prior to accumulator isolation will delay depressurization since accumulator in flow to RCS will tend to stabilize RCS pressure.

TC 96-139

TC 96-140

TC 95-139

## EXAM QUESTION HISTORY

Question # 12      RO \_\_\_\_\_      SRO 12  
                                  TIER 1      Group 2  
B000.1064      KA W/E09EA2.2      Importance 3.8

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

_____	New
<u>X</u>	Modified (Attach original and Modified Questions)
	Original Bank <u>INPO</u> # <u>Diablo Canyon 1/1/2000</u>
	Bank Originating Bank _____ # _____

10CFR55 Content      55.41 \_\_\_\_\_      55.43.5 \_\_\_\_\_

Learning Objective: RES03C 2.01

Given a set of plant and equipment conditions evaluate the conditions to determine the applicable procedure, and from the procedure determine the appropriate EXPECTED ACTIONS or RESPONSE NOT OBTAINED instructions to implement.(ES-0.3)

Cognitive Level      Memory or Fundamental Knowledge      \_\_\_\_\_  
                                  Comprehension or Analysis        X  

Technical Reference ES-0.3

Level of Difficulty (from attachment 3) \_\_\_\_\_

References required on Exam (Attach copy to this attachment)

ES-0.3  
 \_\_\_\_\_  
 \_\_\_\_\_

Answer Analysis: A is incorrect - although these are steps taken during the depressurization, they do not apply to rising PRZR level.  
 \_\_\_\_\_  
 \_\_\_\_\_



Possible 12

**Question**  
Due to a reactor trip and subsequent loss of offsite power, procedure E-0.4, "Natural Circulation Cooldown with Steam Void in Vessel (without RVLIS)," step 8, is in progress. During depressurization to the target pressure, PZR level rapidly increases to 92%.  
What action(s) should the operators take?

**Answer:**  
Stop RCS depressurization, then energize pressurizer heaters to increase RCS pressure by 100 psig.

**Distracter 1**  
Stop RCS depressurization, then isolate accumulators if RCS pressure is less than 1000 psig.

**Distracter 2**  
Continue RCS depressurization until RCS pressure is within 50 psig of the average pressure of all SGs.

**Distracter 3**  
Continue RCS depressurization until RCS pressure is less than 900 psig, then isolate accumulators.

**Distracter Analysis:**

**Answer:**

**Distracter 1:**

**Distracter 2:**

**Distracter 3:**

Question 12 B000.1064

(1 point(s))

Due to a reactor trip and subsequent loss of offsite power, procedure ES-0.3, "Natural Circulation Cooldown with Steam Void in Vessel" is in progress at step 8 Controlling PRZR Level. During depressurization, the PRZR level rapidly increases to 93%.

What action(s) is required?

- a. Stop RCS depressurization, then isolate SI accumulators if RCS pressure is less than 1500 psig.
- b. Stop RCS depressurization, then energize pressurizer heaters to increase RCS pressure by 100 psig.
- c. Continue RCS depressurization until RCS pressure is within 50 psig of average S/G pressure.
- d. Continue RCS depressurization until RVLIS level is less than 90%, then stop depressurization while continuing with cooldown.

Answer 12

- b. Stop RCS depressurization, then energize pressurizer heaters to increase RCS pressure by 100 psig.

## EXAM QUESTION HISTORY

Question #                      RO \_\_\_\_\_                      SRO 13  
                                                  TIER 2                                              Group 1  
B000.1065                      KA 004G2.1.33                                              Importance 4.0

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

<u>  X  </u>	New
_____	Modified (Attach original and Modified Questions)
_____	Original Bank _____ # _____
_____	Bank Originating Bank _____ # _____

10CFR55 Content                      55.41 \_\_\_\_\_                      55.43 2

Learning Objective: R1601C 5.02

Given a set of plant conditions and a copy of TRM and TRM referenced material, be able to apply the TRM. To include: LCO, applicability, applying action items and surveillances.

Cognitive Level                      Memory or Fundamental Knowledge                      \_\_\_\_\_  
                                                  Comprehension or Analysis                                                X  

Technical Reference   TRM 3.1.1  

Level of Difficulty (from attachment 3) \_\_\_\_\_

References required on Exam (Attach copy to this attachment)

  TRM 3.1.1    
 \_\_\_\_\_  
 \_\_\_\_\_

Answer Analysis:   B is correct since TRM 3.1.1 requires two charging pumps to be operable to meet the spec. Actions A and B as stated are applicable.    
 \_\_\_\_\_  
 \_\_\_\_\_



Question 13 B000.1065

(1 point(s))

During normal at power operation with the "B" Charging Pump OOS, the "A" Charging Pump trips and cannot be restarted. Which of the following states the required action.

- a. No Actions Required, Charging Pumps are not Tech Spec required equipment.
- b. Restore a second charging pump to operable status within 72 hours, then be in Mode 3 and verify adequate SDM in 6 hours.
- c. Restore a second charging pump to operable status within 7 days, then be in Mode 5 in 30 hours.
- d. Restore a second charging pump to operable status immediately and be in Mode 3 and verify adequate SDM in 6 hours.

Answer 13

- b. Restore a second charging pump to operable status within 72 hours, then be in Mode 3 and verify adequate SDM in 6 hours.





Question 14 B000.1066

(1 point(s))

During power operations AOV-431A Pressurizer Spray Valve opens to control pressure and becomes mechanically stuck at 50% open. Which one of the following describes the plant response with no operator actions and the operator action to mitigate this event.

- a. RCS pressure will decrease to approximately 1050 psig. Rx trip and SI will occur. Pressure will stabilize at approximately 1050 psig. The operators should trip the Rx and "A" RCP per AP-PRZR.1, Abnormal PRZR Pressure.
- b. RCS Pressure will decrease to below SI pump shutoff head. The Rx will Trip and SI will occur. RCS pressure will stabilize at approximately 1500 psig. The operators should Trip the Rx and "B" RCP per AP-PRZR.1, Abnormal PRZR Pressure.
- c. RCS Pressure will decrease to approximately 1050 psig. Rx Trip and SI will occur. Pressure will stabilize at approximately 1050 psig. The operators should Trip the Rx and "B" RCP per AP-PRZR.1, Abnormal PRZR Pressure.
- d. RCS Pressure will decrease to below SI pump shutoff head. The Rx will Trip and SI will occur. RCS Pressure will stabilize at approximately 1500 psig. The operators should trip the Rx and "A" RCP per AP-PRZR.1, Abnormal PRZR Pressure.

Answer 14

- d. RCS Pressure will decrease to below SI pump shutoff head. The Rx will Trip and SI will occur. RCS Pressure will stabilize at approximately 1500 psig. The operators should trip the Rx and "A" RCP per AP-PRZR.1, Abnormal PRZR Pressure.





Possible #15

QUESTION ID: B022.0004      POINTS: 1      STATUS:      COG LVL:

QUESTION

The shift supervisor has declared the "A" containment recirculation fan cooler (CRFC) inoperable for routine maintenance. Four hours later the "C" CRFC shuts down due to a breaker failure. What are the plant operational restrictions due to these events? (TSAS: Tech Spec Action Statement)

- a. The plant is in a 72-hour TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 84 hours.
- b. The plant is in a 72-hour TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 36 hours.
- c. The plant is in a 7-day TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 36 hours.
- d. The plant is in a 7-day TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 84 hours.

ANSWER

- a. The plant is in a 72-hour TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 84 hours.

Handouts:  
None

Lessons	Enabling Objs.	Tasks	References	K and A's
R2201C	R2201C 6.02	None	None	None

A & C supply Post-Accident charcoal system

Question 15 B022.0004

(1 point(s))

The plant is operating at 100% power with the following equipment alignment: B, C, and D Containment Recirculation Fan Coolers (CRFCs) are in service. All other equipment is available and aligned for normal operation. The "C" CRFC develops high vibration, is declared "inoperable, and is secured after "A" CRFC is started. Five minutes after starting "A" CRFC, it trips on overcurrent. A local reset is attempted. The "A" CRFC will not start.

What are the plant operational restrictions due to these events? (TSAS: Tech Spec Action Statement)

- a. The plant is in a 72-hour TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 84 hours.
- b. The plant is in a 7-day TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 36 hours.
- c. The plant is in a 1-hour TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 48 hours.
- d. The plant is not in a TSAS for required shutdown since both trains of containment spray operable.

Answer 15

- a. The plant is in a 72-hour TSAS, if not met must be in Mode 3 in 6 hours and Mode 5 in 84 hours.

## EXAM QUESTION HISTORY

Question #                      RO \_\_\_\_\_                      SRO 16  
                                          TIER 2                                      Group 1  
B300.0034                      KA 064G2.1.33                      Importance 4.0

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

_____	New
_____	Modified (Attach original and Modified Questions)
_____	Original Bank _____ # _____
<u>X</u>	Bank Originating Bank <u>Ginna</u> # <u>B300.0034</u>

10CFR55 Content                      55.41 \_\_\_\_\_                      55.43 2

Learning Objective: R0801C 10.02

Given a set of plant conditions and a copy of Tech Specs and Tech Specs referenced material i.e. (COLR) be able to apply the Tech Specs. To include: LCO, applicability, applying actions, surveillances and the use of basis to aid in application.

Cognitive Level                      Memory or Fundamental Knowledge                      \_\_\_\_\_  
                                          Comprehension or Analysis                                      X

Technical Reference PT-12.1/TS 3.8.1

Level of Difficulty (from attachment 3) \_\_\_\_\_

References required on Exam (Attach copy to this attachment)

PT-12.1 TS 3.8.1

---

AnswerAnalysis: B is correct - PT-12.1 requires the D/G be declared inoperable due to low lube oil pressure < 68 psig. TS 3.8.1 allows 7 days to repair.

---



Question 16 B300.0034

(1 point(s))

At 97% power during PT-12.1 "Emergency Diesel Generator 1A" test, the following conditions were noted:

Load 2005 KW  
Power Factor .95 Lagging  
Fuel Oil Press 38 psig  
Lube Oil Press 67 psig  
Jacket Water Press 34 psig  
Air Manifold 22 psig

Based on the given information, which one of the following describes any required actions, assuming the "B" D/G and offsite power verified operable.

- a. All requirements satisfied, No Actions.
- b. Declare the "A" D/G inoperable, continued operation allowed for 7 days.
- c. Declare the "A" D/G inoperable, continued operation allowed for 24 hours.
- d. One D/G parameter in the Alert Range. Submit an Action Report.

Answer 16

- b. Declare the "A" D/G inoperable, continued operation allowed for 7 days.





Question 17 B000.1067

(1 point(s))

The following conditions exist:

There is a core off-load in progress

The fuel handlers were moving irradiated fuel to a location in the spent fuel pool

You are notified that the spent fuel bundle was accidentally dropped in the spent fuel pool

The bundle fell into the correct location

R-5, Spent Fuel Pool radiation monitor reads 4 mrem/hr and is steady

What actions, if any, are required?

- a. No action is required
- b. Enter procedure EPIP 1-13, High Radiation Area. Confer with RP about evacuating personnel from the area.
- c. Enter procedure RF-65.4, Fuel Handling Accident. Evacuate personnel from the area and refer to EPIP 1-13, Local Radiation Emergency and EPIP 1-0 Event Classification.
- d. Enter procedure RF-65.4, Fuel Handling Accident. Instruct the bridge to verify proper location in fuel pool and instruct RP to measure radiation levels above pool. Inform reactor engineering.

Answer 17

- c. Enter procedure RF-65.4, Fuel Handling Accident. Evacuate personnel from the area and refer to EPIP 1-13, Local Radiation Emergency and EPIP 1-0 Event Classification.





Question 18 B000.1068

(1 point(s))

During 100% power operations a small fire occurs in the Control Room kitchen. Large quantities of smoke fill the Control Room forcing the operating crew to evacuate. Assuming the fire is limited to the kitchen area, which one of the following procedure(s) provides guidance for operating the plant under these conditions.

- a. AP-CR.1, Control Room Evacuation and ER-FIRE.1, Alternate Shutdown for Control Complex Fire
- b. ER-FIRE.1, Alternate Shutdown for Control Complex Fire
- c. ER-FIRE.0, Control Room Response to Fire Alarm and Reports
- d. AP-CR.1, Control Room Evacuation

Answer 18

- d. AP-CR.1, Control Room Evacuation





Question 19 B006.0001

(1 point(s))

A plant heatup was in progress, with the RCS at 300 degrees F and 350 psig, when the B SI pump failed its surveillance and was declared inoperable. Which one of the below limits are placed on the plant heatup?

- a. All ECCS LCOs satisfied, may operate in any mode.
- b. ECCS LCO for this mode satisfied but may not proceed into mode 3.
- c. ECCS LCO not satisfied. 1 hour to restore B SIP then 24 hrs to mode 5.
- d. ECCS LCO satisfied for this mode but enter a 72 hour clock when proceed into mode 3.

Answer 19

- b. ECCS LCO for this mode satisfied but may not proceed into mode 3.

## EXAM QUESTION HISTORY

Question # B000.1070      RO \_\_\_\_\_      SRO 20  
 TIER 3      Group \_\_\_\_\_  
 KA 2.1.14      Importance 3.3

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

<u>X</u>	New
_____	Modified (Attach original and Modified Questions)
_____	Original Bank _____ # _____
_____	Bank Originating Bank _____ # _____

10CFR55 Content      55.41 \_\_\_\_\_      55.43 5

Learning Objective: RSC08C 3.00

Using appropriate procedures and given a set of plant conditions, the SRO should determine the response of the Control Room for fighting a fire in the Power Block, or on-site within the perimeter of the fence.

Cognitive Level      Memory or Fundamental Knowledge      X  
 Comprehension or Analysis      \_\_\_\_\_

Technical Reference ER-FIRE.0 Appendix A

Level of Difficulty (from attachment 3) \_\_\_\_\_

References required on Exam (Attach copy to this attachment)

None

---

Answer Analysis: B is correct since the Fire Response procedure requires notification of the plant personnel via the paging system.



Question 20 B000.1070

(1 point(s))

A fire alarm on FCP-1 in the Control Room is received. After notifying the Fire Brigade Captain, what is the next action the Control Room is required to take?

- a. Notify RG&E Energy Operations of the fire location and severity.
- b. Notify plant personnel on the plant paging system.
- c. Notify the NRC per O-9.3, NRC Immediate Notification.
- d. Notify New York State, Monroe and Wayne Counties per EPIP 1-5.

Answer 20

- b. Notify plant personnel on the plant paging system.





Question 21 B000.1071

(1 point(s))

Plant in Mode 1, 100% power, steady state conditions. Operators are filling the A SI Accumulator per S-16.13 with the B SI pump when the B SI pump breaker trips open. Which of the following is the correct operator action?

- a. Attempt one B SI pump breaker reset and reclosure.
- b. Reset the B SI pump breaker and start the C SI pump.
- c. Do not attempt reset and reclosure of the B SI pump breaker. Initiate corrective action per IP-CAP.1
- d. Reset the B SI pump breaker and fill the A SI Accumulator with the SI Accumulator makeup pump

Answer 21

- c. Do not attempt reset and reclosure of the B SI pump breaker. Initiate corrective action per IP-CAP.1





Question 22 B300.0061

(1 point(s))

Equipment covered in section 3 of ITS has become inoperable and is being documented by an A-52.4 Attachment. Which one of the following explains when the "Tracking Purposes Only" space should be checked?

- a. The redundant component/train is operable
- b. A WR/TR or WO is not required
- c. At the discretion of the individual completing the attachment.
- d. The equipment is not required to be operable for the present mode.

Answer 22

- d. The equipment is not required to be operable for the present mode.

TC 96-020

### EXAM QUESTION HISTORY

Question # C000.1061 RO \_\_\_\_\_ SRO 23  
TIER 3 Group \_\_\_\_\_  
KA 2.3.6 Importance 3.1

Source of Question (Note: Attach question and any subsequent modifications to this attachment)

_____	New
_____	Modified (Attach original and Modified Questions)
_____	Original Bank _____ # _____
<u>X</u>	Bank Originating Bank <u>Ginna</u> # <u>C000.1061</u>

10CFR55 Content 55.41 \_\_\_\_\_ 55.43 4 \_\_\_\_\_

Learning Objective

\_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

Cognitive Level Memory or Fundamental Knowledge X  
Comprehension or Analysis \_\_\_\_\_

Technical Reference CH-RETS-GDT-REL

Level of Difficulty (from attachment 3) \_\_\_\_\_

References required on Exam (Attach copy to this attachment)

None

\_\_\_\_\_

Answer Analysis: A is correct since the release procedure requires that R-14/R-14A be checked prior to a GDT release.



Question 23 C000.1061

(1 point(s))

Which of the following items is the Shift Supervisor required to verify prior to authorizing a Gas Decay Tank Release?

- a. Radiation Monitors R-14 or R14A operable
- b. Both Plant Vent Exhaust Fans are running
- c. The Gas Decay Tank has been isolated and held for at least 30 days
- d. The release will be initiated within 24 hours of tank sample time.

Answer 23

- a. Radiation Monitors R-14 or R14A operable





Question 24 B000.1069

(1 point(s))

Given the following conditions:

Plant is in Mode 5

Irradiated Fuel Movement is in progress inside CNMT

CNMT Purge is in service

R-12A is operable and in service

R-12 Containment Noble Gas Monitor fails

Using the attached references, which of the following statements describes the required action for this failure.

- a. 4 hours to repair R-12, then remove purge from service and close the purge supply and exhaust valves.
- b. Stop movement of Irradiated Fuel immediately, CNMT Purge may continue
- c. No actions are required since R-12A is in service and can be used to monitor CNMT purge.
- d. Purge may continue provided an operator is dedicated to monitoring R-12A and securing CNMT purge if R-12A alarms.

Answer 24

- a. 4 hours to repair R-12, then remove purge from service and close the purge supply and exhaust valves.





Question 25 B000.1072

(1 point(s))

While the plant is at 100% power, the Primary AO notifies the Control Room that he found SI pump suction valve V-890A with its chain cut and in the closed position. After notifying Security of this event, which of the following actions are the operators required to perform?

- a. Begin a Plant Shutdown per O-2.1, Shutdown to Hot Shutdown Conditions, continuing until SI Train A components are verified in their correct position.
- b. Perform the S-30 Safeguard Equipment Lineup check procedures to verify correct system valve and breaker positions.
- c. Send two qualified operators to perform an independent verification on V-890A and to relock the valve.
- d. Order the evacuation of non-essential plant personnel to the Training Center until an investigation of the incident is complete.

Answer 25

- b. Perform the S-30 Safeguard Equipment Lineup check procedures to verify correct system valve and breaker positions.