

November 17, 2004

LICENSEE: Nuclear Management Company, LLC

FACILITY: Point Beach Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON NOVEMBER 17, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference on November 17, 2004, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application. The conference call was useful in clarifying the intent of the staff's D-RAIs.

Enclosure 1 provides a listing of the meeting participants. Enclosure 2 contains a listing of the D-RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/

Michael J. Morgan, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-266 and 50-301

Enclosures: As stated

cc w/encls: See next page

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DISTRIBUTION: To Licensee: Nuclear Management Co., LLC, Re: Pt. Beach Nuclear Plant, Units 1 and 2, Dated: November 17, 2004

Adams accession no.: **ML043270652**

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TO DISCUSS THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
NOVEMBER 17, 2004

Participants

Affiliations

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DRAFT REQUESTS FOR ADDITIONAL INFORMATION (D-RAI)
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION

November 17, 2004

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on November 17, 2004, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal application (LRA). The following D-RAIs were discussed during the telephone conference call.

D-RAI-2.1 .1 Short Term Exposure Duration Definition - 10 CFR 54(a)(2)

The PBNP LRA and page 13 of LR-TR-514 did not adequately define short term exposure duration for low and moderate energy piping failures covered under 10 CFR 54.4(a)(2) that could affect safety related electrical equipment under the scope of 10 CFR 54.4(a)(1). Specifically, the staff found that some safety-related electrical equipment may exist in the turbine building or other parts of the plant and may be subject to harsh environments from low or moderate energy pipe breaks but are not environmentally qualified (EQ). Since this equipment may not be EQ, they could fail due to 10 CFR 54.4(a)(2) piping failures.

The staff requests additional information to adequately define short term exposure duration for low and moderate energy piping failures and how it relates to scoping and screening of 10 CFR 54.4(a)(2) piping that could cause these types of failures.

Discussion: Based on the discussion with the applicant, the staff indicated and the applicant agreed that this question required clarification. Applicant will clarify information/response.

D-RAI- 2.1.2 First Equivalent Anchor Definition - 10 CFR 54(a)(2)

The PBNP LRA Section 2.1.2.1.2, page 2-19, states, under, NSR SSCs Directly Connected to SR SSCs, "For NSR SSCs directly connected to SR SSCs (typically piping systems), the NSR piping and supports, up to and including the first equivalent anchor beyond the safety/non safety interface, are within the scope of license renewal per 10 CFR 54.4(a)(2). Although these piping segments are not uniquely identified on the LR boundary drawing, applicable aging effects on these piping segments are managed along with the adjoining SR piping."

The staff requests additional information to adequately describe and define what is meant by the first equivalent anchor and how it relates to the scoping and screening of 10 CFR 54.4(a)(2) NSR piping and supports.

Discussion: Based on the discussion with the applicant, the staff indicated and the applicant agreed that this question required clarification. Applicant will clarify information/response.

RAI -2.1.3 Flow Accelerated Corrosion affect on Piping Section Scoping - 10 CFR 54(a)(2)

The PBNP LRA Section 2.1.2.1.2, pages 2-20&21, states, under Piping Supports, ""All NSR supports for non-seismic or Seismic II/I piping systems with a potential for spatial interaction with safety related SSC, will be included within the scope of license renewal per 10 CFR 54.4(a)(2). These supports will be addressed in a commodity fashion, within the civil/structural area review. As long as the effects of aging on the supports for these piping systems are managed, falling of piping sections, except for flow accelerated corrosion (FAC) failures, is not considered credible, and the piping section itself would not be in-scope for 10 CFR 54.4(a)(2) due to physical impact hazard (although the leakage or spray may still apply)."

The staff requests additional information to adequately describe why the falling of piping sections is not considered credible, and why the piping section itself would not be in-scope for 10 CFR 54.4(a)(2) due to physical impact hazard. Please describe how the management of FAC relates to the scoping and screening of 10 CFR 54.4(a)(2) Seismic II/I piping systems that could cause these types of failures.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.