

November 17, 2004

Mr. Dennis L. Koehl  
Site Vice President  
Nuclear Management Company, LLC  
6610 Nuclear Road  
Two Rivers, WI 54241

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL  
APPLICATION

Dear Mr. Koehl:

By letter dated February 25, 2004, Nuclear Management Company, LLC, (NMC or the applicant) submitted an application pursuant to 10 CFR Part 54, to renew the operating licenses for Point Beach Nuclear Plant (PBNP), Units 1 and 2, for review by the U.S. Nuclear Regulatory Commission (NRC). The NRC staff is reviewing the information contained in the license renewal application (LRA) and has identified, in the enclosure, areas where additional information is needed to complete the review.

These RAs were discussed with your staff, Mr. Jim Knorr, and a mutually agreeable date for this response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-2232 or e-mail [MJM2@nrc.gov](mailto:MJM2@nrc.gov).

Sincerely,  
**/RA/**

Michael J. Morgan, Project Manager  
License Renewal Section A  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket Nos.: 50-266 and 50-301

Enclosure: As stated

cc w/encls: See next page

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OFFICE	PM:RLEP	SC:RLEP
NAME	MMorgan	SLee
DATE	11 /17/04	11/17/04

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Point Beach Nuclear Plant, Units 1 and 2

cc:

Jonathan Rogoff, Esq.  
Vice President, Counsel & Secretary  
Nuclear Management Company, LLC  
700 First Street  
Hudson, WI 54016

Mr. Frederick D. Kuester  
President and Chief Executive Officer  
We Generation  
231 West Michigan Street  
Milwaukee, WI 53201

James Connolly  
Manager, Regulatory Affairs  
Point Beach Nuclear Plant  
Nuclear Management Company, LLC  
6610 Nuclear Road  
Two Rivers, WI 54241

Mr. Ken Duveneck  
Town Chairman  
Town of Two Creeks  
13017 State Highway 42  
Mishicot, WI 54228

Chairman  
Public Service Commission  
of Wisconsin  
P.O. Box 7854  
Madison, WI 53707-7854

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
801 Warrenville Road  
Lisle, IL 60532-4351

Resident Inspector's Office  
U.S. Nuclear Regulatory Commission  
6612 Nuclear Road  
Two Rivers, WI 54241

Mr. Jeffrey Kitsembel  
Electric Division  
Public Service Commission of Wisconsin  
P.O. Box 7854  
Madison, WI 53707-7854

David Weaver  
Nuclear Asset Manager  
Wisconsin Electric Power Company  
231 West Michigan Street  
Milwaukee, WI 53201

John Paul Cowan  
Executive Vice President & Chief Nuclear  
Officer  
Nuclear Management Company, LLC  
700 First Street  
Hudson, WI 54016

Douglas E. Cooper  
Senior Vice President - Group Operations  
Palisades Nuclear Plant  
Nuclear Management Company, LLC  
27780 Blue Star Memorial Highway  
Covert, MI 49043

Fred Emerson  
Nuclear Energy Institute  
1776 I Street, NW., Suite 400  
Washington, DC 20006-3708

Roger A. Newton  
3623 Nagawicka Shores Drive  
Hartland, WI 53029

James E. Knorr  
License Renewal Project  
Nuclear Management Company, LLC  
6610 Nuclear Road  
Point Beach Nuclear Plant  
Two Rivers, WI 54241

Dennis L. Koehl  
Site Vice President  
Point Beach Nuclear Plant  
Nuclear Management Company, LLC  
6610 Nuclear Road  
Two Rivers, WI 54241

DISTRIBUTION: Ltr. To D. Koehl, Request for RAI for the Review of the Pt. Beach Nuclear Plant, Units 1 & 2, Dated: November 17, 2004

Adams accession no.: **ML043270650**

**HARD COPY**

RLEP RF  
Project Manager

**E-MAIL:**

RidsNrrDrip  
RidsNrrDe  
G. Bagchi  
K. Manoly  
W. Bateman  
J. Calvo  
R. Jenkins  
P. Shemanski  
J. Fair  
RidsNrrDssa  
RidsNrrDipm  
D. Thatcher  
R. Pettis  
G. Galletti  
C. Li  
M. Itzkowitz (RidsOgcMailCenter)  
R. Weisman  
M. Mayfield  
A. Murphy  
S. Smith (srs3)  
S. Duraiswamy  
Y. L. (Renee) Li  
RLEP Staff  
-----

L. Kozak, RIII  
P. Loughheed, RIII  
J. Strasma, RIII  
A. Vogel, RIII  
H. Chernoff  
W. Ruland  
C. Marco  
L. Raghavan  
T. Mensah  
OPA

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2  
LICENSE RENEWAL APPLICATION (LRA)  
REQUEST FOR ADDITIONAL INFORMATION (RAI)

**4.4 Fracture Mechanics Analysis**

**RAI 4.4.2 (Reactor Coolant Pump Flywheel Analysis)**

The staff needs explanation as to the number of reactor coolant pump (RCP) start/stop cycles that are assumed in the 60-year RCP flywheel fatigue crack growth assessment for the Point Beach Units.

**B 2.0 Aging Management Programs**

**RAI B 2.1.6-1 (Boric Acid Corrosion Program)**

The staff seeks additional clarification regarding the list of components that are within the scope of the Boric Acid Corrosion Program and the process the applicant uses to augment the list of components within the scope of the AMP based on pertinent industry experience. This is RAI B2.1.6-1. Specifically, the staff requested the following actions of the applicant:

- C Submittal of a discussion on how NMC's responses to NRC Bulletin 2002-01, dated March 29, 2002, and May 16, 2002; responses to the NRC's RAIs on the bulletin, dated January 17, 2003; response to NRC Bulletin 2003-02, dated September 19, 2003; responses to NRC Order EA-03-009, dated March 3, 2003, April 11, 2003, and April 18, 2003; and response to NRC Bulletin 2004-01, dated May 28, 2004 have been used to update the list of component locations and types of visual inspections credited within the scope of the Boric Acid Corrosion Program or within the scope of other aging management programs (AMPs) that provide for implementation of similar or more conservative types of inspections.
- C If the responses were used to supplement the scope of the Boric Acid Corrosion Program or other AMPs, identification of the component locations that have been added to the scope of the program and clarification of the type of visual examinations (i.e., specification on whether VT-1, VT-2 or VT-3 will be used and whether the visual examinations will be enhanced, bare-surface, qualified, etc.) that will be implemented on those components within the current scope of the program.

**RAI B 2.1.18-1 (Reactor Vessel Surveillance Capsule Program)**

GALL Program XI.M31 suggested that standby capsules are to be removed and placed in storage. Even though the capsules do not contain limiting material, these standby capsules provide general embrittlement trends and provide assurance that current embrittlement methodologies apply to Point Beach. Leaving the capsules in the vessel, further exposure would not provide any meaningful data. Please justify your decision of not removing the capsule and keep it in storage.

Enclosure

#### RAI B 2.1.23-1 (Thimble Tube Inspection Program)

Section B2.1.23 indicates that eddy current examinations are performed on a periodicity consistent with the severity of wear damage for each thimble tube. The frequency of inspections is based on the maximum wall loss noted in a region of active wear and the projected wear which would occur based on a known wear rate.

1. Identify the wear rate that is currently being used and how did you calculate the wear rate. Based on this wear rate, how were the inspection intervals determined to ensure that wear resulting from flow induced vibration does not result in the wall thickness below the minimum required thimble tube integrity?
2. Specify the NDE uncertainty that is used in the calculations along with a justification for the NDE uncertainty value assumed in the calculation. Note that we would like the NDE uncertainty to be specified as a given percentage of the nominal wall thickness for the thimble tubes.

The applicant's Operating Experience identified certain problems related to inspection deferrals, calculation methodology, and record retention.

Explain the problems in detail and how and when are you going to address the issues.