

November 8, 2004

Mr. R. T. Ridenoure  
Division Manager - Nuclear Operations  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
Post Office Box 550  
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOON STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT  
(TAC NO. MC3233)

Dear Mr. Ridenoure:

The Commission has issued the enclosed Amendment No. 230 to Renewed Facility Operating License No. DPR-40 for the Fort Calhoun Station, Unit No. 1. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated May 14, 2004.

The amendment relocates the requirements of TS 3.3(1)a, "Reactor Coolant System and Other Components Subject to ASME XI Boiler & Pressure Vessel Code Inspection and Testing Surveillance" and TS 3.4, "Reactor Coolant System Integrity Testing," to the Updated Safety Analysis Report (USAR). The requirements in TS 3.3(1)a were related to inservice inspection of ASME Class 1, 2, and 3 components and requirements in TS 3.4 were related to reactor coolant system integrity testing.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

**/RA/**

Alan B. Wang, Project Manager, Section 2  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures: 1. Amendment No. 230 to DPR-40  
2. Safety Evaluation

cc w/encls: See next page

Ft. Calhoun Station, Unit 1

cc:

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(TAC NO. MC3233)

Dear Mr. Ridenoure:

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The amendment relocates the requirements of TS 3.3(1)a, "Reactor Coolant System and Other Components Subject to ASME XI Boiler & Pressure Vessel Code Inspection and Testing Surveillance" and TS 3.4, "Reactor Coolant System Integrity Testing," to the Updated Safety Analysis Report (USAR). The requirements in TS 3.3(1)a were related to inservice inspection of ASME Class 1, 2, and 3 components and requirements in TS 3.4 were related to reactor coolant system integrity testing.

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2. Safety Evaluation

cc w/encls: See next page

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OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 230  
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Omaha Public Power District (the licensee) dated May 14, 2004, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Renewed Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 230, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 120 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

Robert A. Gramm, Chief, Section 2  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: November 8, 2004

ATTACHMENT TO LICENSE AMENDMENT NO. 230

RENEWED FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

REMOVE

3.3 - Page 1

3.4 - Page 1

INSERT

3.3 - Page 1

3.4 - Page 1

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 230 TO RENEWED FACILITY

OPERATING LICENSE NO. DPR-40

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

1.0 INTRODUCTION

By application dated May 14, 2004, Omaha Public Power District (OPPD) requested changes to the Technical Specifications (Appendix A to Renewed Facility Operating License No. DPR-40) for the Fort Calhoun Station, Unit No. 1 (FCS). The proposed amendment would relocate the requirements of Technical Specification (TS) 3.3(1)a, "Reactor Coolant System and Other Components Subject to ASME XI Boiler & Pressure Vessel Code Inspection and Testing Surveillance" and TS 3.4, "Reactor Coolant System Integrity Testing" to the FCS Updated Safety Analysis Report (USAR). Requirements in TS 3.3(1)a were related to inservice inspection of ASME Class 1, 2, and 3 components and requirements in TS 3.4 were related to reactor coolant system integrity testing.

2.0 REGULATORY EVALUATION

The staff finds that the licensee in Section 4.0 of its submittal identified the applicable regulatory requirements. The regulatory requirements for which the staff based its acceptance are:

- 10 CFR 50.55a(a)(2) states in part that systems and components of boiling and pressurized water-cooled nuclear power reactors must meet the requirements of the ASME Boiler and Pressure Vessel Code specified in paragraphs (b), (c), (d), (e), (f), and (g) of this section.
- Inservice inspection of the ASME Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code (ASME Code) and applicable addenda as required by Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(6)(g)(I).
- 10 CFR 50.36(c)(3) states that surveillance requirements are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation (LCO) will be met. Section 50.36(c)(2)(ii) states that a TS LCO of a nuclear reactor must be established for each item meeting one or more of the following criteria:  
Criterion 1 – Installed instrumentation that is used to detect and indicate in the control

room, a significant abnormal degradation of the reactor coolant pressure boundary.

Criterion 2 – A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

Criterion 3 – A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

Criterion 4 – A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

### 3.0 TECHNICAL EVALUATION

The staff has reviewed the licensee's regulatory and technical analyses in support of its proposed license amendment which are described in Sections 5.0 and 6.0 of the licensee's submittal. The detailed evaluation below will support the conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

The proposed amendment relocates the requirements of TS 3.3(1)a concerning inservice inspection of ASME Code, Class 1, 2, and 3 components and TS 3.4 concerning reactor coolant system integrity testing to the FCS USAR. These surveillance requirements are specified to monitor component degradation and to ensure structural integrity during operation.

10 CFR 50.36(c)(2)(ii) states that a TS LCO of a nuclear reactor must be established for each item meeting one or more of the four criteria as noted in Section 2.0, "Regulatory Evaluation." The staff has reviewed the TSs against the 4 criteria noted in 10 CFR 50.36(c)(2)(ii) and concludes the following:

Criterion 1. The surveillance requirements under the TS are not related to any installed instrumentation used to detect a significant abnormal degradation of the reactor coolant system boundary prior to a design basis accident (DBA).

Criterion 2. The surveillance requirements under the TS do not monitor any process variable or a design feature or any operating restriction that is an initial condition in a DBA or transient analysis.

Criterion 3. The inspection of ASME Code, Class 1, 2, and 3 components is a monitoring activity of component degradation to prevent inservice failure and is, therefore, not necessary to be retained in the TS for immediate operability of safety



systems. Further, the inspections and tests are usually performed during plant shutdown.

Criterion 4. The FCS Probabilistic Safety Assessment does not address inspection and testing of ASME Code, Class 1, 2, and 3 components. From the FCS risk-informed inservice inspection program outline, it is noted that a reduced ASME Section XI inspection coverage is a minor contributor to the core damage frequency or the large early release frequency.

Therefore, based on the staff review, we have concluded that the surveillance requirements that are being proposed to be relocated to the USAR do not meet the screening criteria of 10 CFR 50.36(c)(2)(ii) for inclusion in the TS and can be relocated to the USAR. The relocation of these TSs to the FCS USAR will not change the type or frequency of inspections as currently required by the TSs as the licensee must continue to meet the regulations pursuant to 10 CFR 50.55a with regard to inservice inspection. The relocated requirements of TS 3.3(1)a concerning inservice inspection of ASME Code, Class 1, 2, and 3 components and TS 3.4 concerning reactor coolant system integrity testing will be adequately controlled by 10 CFR 50.59 or 10 CFR 50.55a as applicable. Based on the above and the staff's review of the technical justification provided by the licensee, the staff concludes that the proposed TS change is in accordance with the current licensing basis and is, therefore, acceptable. In addition, the proposed TS as modified by the relocation of surveillance requirements in TS 3.3(1)a and TS 3.4 to the FCS USAR is consistent with the contents of the improved Standard Technical Specifications pursuant to the guidance in the Final Policy Statement on TS improvements for nuclear power reactors and conforms to the standard TS provided in NUREG-1431, Rev. 1, with appropriate modifications for plant-specific considerations.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Nebraska State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (69 FR 34703). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22©(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Patnaik

Date: November 8, 2004