

November 9, 2004

LICENSEE: Indiana Michigan Power Company
FACILITY: Donald C. Cook Nuclear Plant, Units 1 and 2
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON
SEPTEMBER 28, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY
COMMISSION AND INDIANA MICHIGAN POWER COMPANY, PERTAINING
TO THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE
RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the NRC or the staff) and representatives of Indiana Michigan Power Company (I&M) held a telephone conference call on September 28, 2004, to discuss and clarify requests for additional information (RAIs) concerning the Donald C. Cook Nuclear Plant, Units 1 and 2, license renewal application (LRA). The conference call was useful in clarifying the intent of the staff's RAIs.

Enclosure 1 provides a listing of the telephone conference call participants. Enclosure 2 contains the items discussed with the applicant, including a brief description on the status of each item.

The applicant has had an opportunity to comment on this summary.

/RA/

Jonathan G. Rowley, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

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TO DISCUSS THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
SEPTEMBER 28, 2004

Participants

Jonathan Rowley
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**REQUESTS FOR ADDITIONAL INFORMATION
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION**

SEPTEMBER 28, 2004

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Indiana Michigan Power Company (I&M) held a telephone conference call on September 28, 2004, to discuss and clarify requests for additional information concerning the Donald C. Cook Nuclear Plant, Units 1 and 2 (CNP), license renewal application (LRA). The following RAIs were discussed during the telephone conference call.

1. RAI B.1.5-2

LRA Section B.1.5 provides the acceptance criteria of BMI thimble tubes as: (1) replacement or isolation of a thimble tube with 80 percent through-wall wear, (2) reposition of a thimble tube with more than 40 percent through-wall wear, provided that it is projected to remain under 80 percent until the next inspection, and (3) replacement, isolation, or reposition of a thimble tube with more than 40 percent through-wall wear if it is projected to exceed 80 percent by the next inspection. Using reposition as an option for Criterion 3 for a tube which is projected to exceed 80-percent wear by the next inspection is inadequate because the uncertainty of the tube wear rate at the selected location for the tube reposition in a certain time period might make the reposition ineffective. Provide a revision of the AMP by incorporating ET uncertainty in future wear measurements and by considering only replacement and isolation of tubes as options for Criterion 3 of the acceptance criteria.

Additional Information Requested

Please confirm that for thimble tubes that are repositioned, the final relocation position is selected based on plant-specific data.

Status

The applicant indicated that the request is clear. The applicant will submit a supplemental response.

2. RAI B.1.24-2

The spray head and its associated components covered by LRA Section B.1.24 may be subject to severe thermal cycling. Inadequate justification was provided to demonstrate that a VT-3 examination is sufficient to detect a potential flaw in the spray head which could lead to failure of the component. Provide justification for using VT-3 examination instead of VT-1 examination for the one-time inspection of these components in either Unit 1 or Unit 2. In addition, provide information regarding acceptance criteria; the evaluation methodology for disposition of indications; and the need for successive examinations for the one-time inspection of spray head, spray head locking bar, and coupling. Also, please provide your commitment in the commitment list and in the UFSAR Supplement.

Enclosure 2

Additional Information Requested

If the CNP Appendix R evaluation credits the spray pattern provided by the pressurizer spray heads, visual examinations of the spray heads performed to VT-3 examination methods may not be adequate. Please confirm pressurizer spray credited in the CNP Appendix R analysis, and if so, provide the basis for determining VT-3 examinations will be sufficient to ensure the spray heads will be capable of performing their intended functions through the period of extended operation.

Status

The applicant indicated that the request is clear. The applicant will submit a supplemental response.

3. RAI B.1.26-2

GALL Program XI.M31 lists 8 items of consideration for an acceptable reactor vessel surveillance program. Item 4 indicates that all pulled and tested capsules, unless discarded before August 31, 2000, are placed in storage. Please provide information regarding consistency with GALL with respect to this item. Also, the staff noticed that Item 6 indicates that all other standby capsules exceeding equivalent RPV fluence of 60 EFPY are to be removed and placed in storage. Please provide the projected dates (in terms of RPV EFPY) for all standby capsules (Capsules V and Z for Unit 1 and Capsules V, W, and Z for Unit 2) to reach the fluence equivalent to 60 EFPY of RPV fluence and the plan to remove and store these standby capsules.

Status

The applicant indicated that the question is clear. The applicant will submit a supplemental response.

4. RAI B.1.27-1

Because of the limited information provided in LRA Section B.1.27, "Reactor Vessel Internals Plates, Forgings, Welds, and Bolting," the staff could not verify that this program is consistent with GALL for most of the 10 elements. For example, the LRA does not mention the identification of the most susceptible items, an Attribute 1 concern; the specific water chemistry guidelines used, an Attribute 2 concern; and whether enhanced visual VT-1 examinations or ultrasonic testing will be employed in inspections for certain selected components and locations, an Attribute 4 concern. Provide information regarding whether all 10 elements of the program are in accordance with GALL Program XI.M16, "PWR Vessel Internals," and whether your program contains any exceptions or enhancements.

Additional Information Requested

In the response to RAI B.1.27-1, the applicant did not specify the visual inspection technique to be used in the implementation of this AMP. Please confirm that this AMP will include the

Materials Reliability Program (MRP) recommendations, including the appropriate visual inspection technique, to be used for inspections performed under this AMP.

Status

The applicant indicated that the request is clear. The applicant will submit a supplemental response.

5. RAI B.1.27-2

The information provided in LRA Section B.1.27 is insufficient for the staff to determine whether the PWR Materials Reliability Project (MRP) Issues Group and Westinghouse Owners Group (WOG) programs discussed there address all key issues of this aging management program (AMP), i.e., crack initiation and growth due to stress corrosion cracking (SCC) or irradiation-assisted SCC, loss of fracture toughness due to neutron irradiation embrittlement, and distortion due to void swelling. Provide a description of all the tasks under the MRP program and their goals and an assessment of the relevance of these tasks to the three aging effects mentioned above. Provide the same for the WOG program for baffle and former bolting. Further, please include your participation in the MRP program as a commitment in your LRA commitment list and in the UFSAR Supplement. Also, please provide a commitment that the program to manage void swelling will be submitted for staff review and approval three years prior to the period of extended operation.

Additional Information Requested

In the original response to RAI B.1.27-2, I&M revised the Reactor Vessel Internals Plates, Forgings, Welds, and Bolting Program commitment made in the LRA to state that the program to manage void swelling will be submitted for staff review and approval three years prior to the period of extended operation. In a public meeting conducted on October 5, 2004, the NRC Staff clarified the original RAI as follows:

Please provide a commitment that the RVI Plates, Forgings, Welds, and Bolting Program will be submitted for staff review and approval three years prior to the period of extended operation.

Status

The applicant indicated that the request is clear. The applicant will submit a supplemental response.

Donald C. Cook Nuclear Plant, Units 1 and 2

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