Facility: Indian Point 2		Scenario	No.: NRC#1	Op-Test No.: 1
Examiners:		1.2.2.1111	Operators:	

Initial Conditions: 3% Rated Thermal Power, MOL

Turnover: Unit 2 is at 3% power, recovering from a 7 day forced outage to repair body to bonnet leak on PRZR Spray Loop 23 Bypass Valve 524. Shift orders are to continue the startup in accordance with Pop 1.3 Plant Startup, Mode 2 to Mode 1. The previous shift completed POP 1.3 though step 4.23. Operations Manager has directed you to baseload the HP Steam Dumps to approximately 8% prior to placing the unit on line per POP-2.1. The Operations Manager, Reactor Engineering and Power Marketing have authorized a rate of power increase of 200 MWe per hour to 100% RTP.

Event No.	Malf. No.	Event Type*	Event Description
1		N SRO/BOP R RO	Raise reactor power
2	XMT- RCS020A	I ALL	Pressurizer Level Channel 2 (LT-460) Fails Low (TS SRO)
3	MAL- RCS014B	C ALL	22 SG Tube Leak (5 gpm) (TS SRO)
4	MAL- RCP007C	C SRO/BOP	23 RCP High Vibration
5	MAL- ATS007A	C SRO/RO	21 Main Boiler Feed Pump Trip (Manual reactor trip required)
6	MAL- RCS014B MAL- EPS001	M ALL	SGTR with subsequent Loss of Offsite Power
7	MOC- SIS001	C SRO/BOP	21 SI Pump Fails to Auto Start

<sup>\* (</sup>N)ormal (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

#### **Reset to IC-124 Validation time = 105 minutes.**

Execute batch file Bat "NRC#1.bat" from Ph.D Expert window:

^ MOC-SIS001 SI1 SAFETY INJECTION PUMP 21 MOTOR IMF MOC-SIS001 (-1 0) 4

^ XMT-RCS020A FIXED OUTPUT: LT-460 PZR LEVEL CH.2 IMF XMT-RCS020A (1 0) 0.000000 0 40.412601

^ MAL-RCS014B STEAM GENERATOR 22 TUBE LEAK (NR)

IMF MAL-RCS014B (2 0) 0.050000 0 0.000000

^ MAL-RCP007C RCP 23 HIGH VIBRATION

IMF MAL-RCP007C (3 0) 10.000000 7200 4.000000

^ MAL-ATS007A MBFP 21 THRUST BEARING FAILURE (NR)

IMF MAL-ATS007A (40) TRUE

^ Loss of Offsite Power when manual SI pushed

TRGSET 29 "xaoi610a.eq.1"

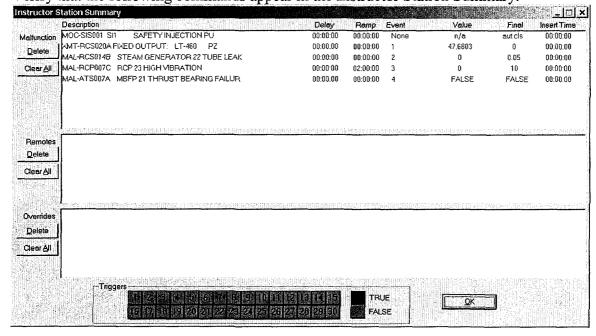
TRG 29 "IMF MAL-EPS001 (-1 0) TRUE"

^ SGTR 22 SG gets big when reactor trips

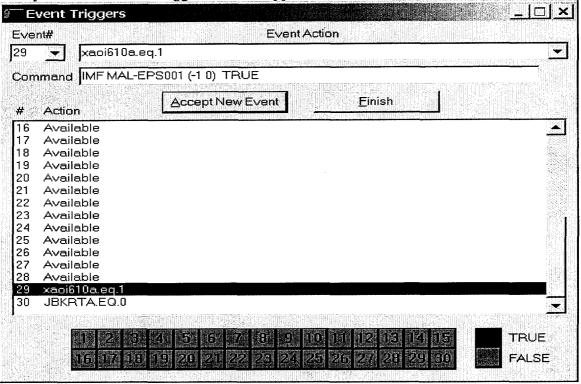
TRGSET 30 "JBKRTA.EQ.0"

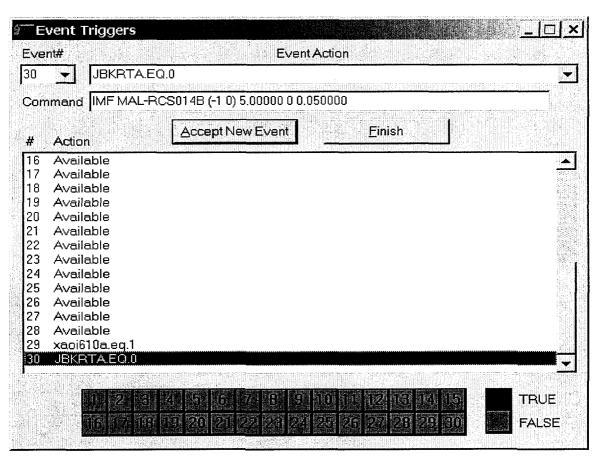
TRG 30 "IMF MAL-RCS014B (-1 0) 5.00000 0 0.050000"

Verify that the following commands appear in the Instructor Station Summary:



Verify that Conditional Trigger 29 & 30 appears on Event Trigger 29 and 30 as follows:





Prior to start of scenario, brief team on power escalation orders. Allow them to formulate a reactivity plan and do their brief outside the simulator. Provide them required graphs and NUPOP, and turnover sheet.

Page 1 of 1 Event No.: 1 Op-Test No.: 1 Scenario No.: 1 Event Description: Raise Reactor Power from 3% reactor power Applicant's Actions or Behavior Time Position SRO Directs activities associated with power increase per 2-POP-1.3, "Plant Startup, Mode 2 to Mode 1": **Rod Motion** Dilution RO Adds Positive reactivity **Rod Motion** Dilution SRO Directs activities associated with shifting Feedwater from Auxiliary Feedwater Pumps to Main Feedwater Pumps BOP Shifts Feedwater from Auxiliary Feedwater Pumps to Main Feedwater Pumps per 2-SOP-21.1 "Main Feedwater System." Checks operation of main feed and bypass feed flow control valves Shifts Feedwater control from AFW pumps to Main Feedwater System Control SG levels on main Feedwater bypass flow control Align AFW pumps and flow control valves for automatic operation per 2-SOP-21.3 "Auxiliary Feedwater Operation" Initiates Turbine Generator startup operations per 2-SOP-26.4. **SRO** "Turbine Generator Startup, Synchronizing, Voltage Control and Shutdown"

#### SIMULATOR OPERATOR:

Activate trigger 1 to start the next event Pressurizer Level Channel 2 (LT-460) Fails Low (TS SRO) when directed by the Lead Evaluator.

Op-Test No.: 1 Scenario No.: 1 Event No.: 2 Page 1 of 3 Event Description: Pressurizer Level Channel 2 (LT-460) Fails Low

Time	Position	Applicant's Actions or Behavior	
	RO	Diagnose Pressurizer Level Channel 2 failed low	
	BOP	Refers to Alarm response Procedure - SAF 3-3 Pressurizer Low Level 18% 5% - SAF 4-3 Pressurizer Lo Lo Level Channel Trip 5% - SGF 2-9 RCS Reduced Inventory - SFF 1-7 PRZR Heater Group Tripped	
	SRO	Directs RO to perform immediate operator actions (from memory) of 2-AOP-INST-1, "Instrument/Controller Malfunctions"	
	RO	Places 22 Charging Pump speed controller in manual and adjusts charging flow as necessary to maintain Pressurizer Level on program (per Graph RCS-2, "Pressurizer Level vs. Tave")	
	SRO	Directs actions to select operable channels for control and alarm	
	ВОР	Defeats controlling PRZR Level Channel by placing L/460A (Pressurizer Level Defeat Transfer Switch) in DEFEAT CH2 (located in Foxboro rack B-6)	
	SRO	Directs initiation of actions to restore letdown using SOP-3.1, "Charging Seal Water and Letdown Control"	
	RO	Restores letdown:	
		- Verifies a charging pump is in service	
#CCCCCC		<ul> <li>Verifies HCV-142, Charging Line Flow Controller is throttled open</li> </ul>	
		<ul> <li>Verifies Charging flow established with 204A 22 Hot Leg Alternate Charging Stop is OPEN</li> </ul>	
		- Verifies letdown orifice stops CLOSED (200 A, B, C)	
		<ul> <li>Verifies that switch Letdown Flow Control Valves ABC in REMOTE</li> </ul>	
		- Verifies Letdown Line Isolation Stops 200A and 200B are OPEN	

Op-Test No.: 1 Scenario No.: 1 Event No.: 2 Page 2 of 3

Event Description: Pressurizer Level Channel 2 (LT-460) Fails Low

Time	Position	Applicant's Actions or Behavior	
	RO	Restores letdown (continued):	
		- OPENS LCV 459 Letdown Stop and places control switch to AUTO	
		<ul> <li>Places PCV-135, Low Pressure Letdown Line Controller to manual and adjusts to 50%</li> </ul>	
		OPENs desired letdown orifice stop to obtain desired letdown flow (200A or 200C excepted to obtain 75 gpm letdown flow)	
		- ADJUSTs PCV-135 to maintain 225-275 psig Letdown Pressure	
		- Return PCV-135 to AUTO	
	SRO	Direct actions to restore backup heaters to pre-failure configuration (21 AUTO, 22 and 23 ON) (or as desired by SRO)	
	RO	Reset PRZR Backup heaters by taking control switches for each group (21, 22, and 23) to OFF and then returning to AUTO or ON as directed by SRO	
	SRO	Direct actions to reset modulating heaters	
	RO	Resets Modulating Heaters:	
		<ul> <li>Place Modulating Heaters control switch in TRIP and then back to neutral.</li> </ul>	
		- Place control switch in CLOSE and then back to neutral	
	SRO	Direct actions to place 22 Charging Pump in AUTO (if desired)	

		cenario No.: 1 Event No.: 2 Page 3 of 3 essurizer Level Channel 2 (LT-460) Fails Low
Time	Position	Applicant's Actions or Behavior
Time		
	RO	Places 22 Charging Pump to AUTO:
		- Place speed control to MAN-BAL
		- Adjust bias knob until deviation meter indicates zero
		- Place speed controller to AUTO
		- Slowly return bias knob to zero
		- Verify RCP Seal Injection Flow 6-12 GPM
		- Monitor automatic Przr level control per Graph RCS-2,  "Pressurizer Level vs Tave"
	SRO	Refer to Technical Specification 3.3 "Instrumentation" and Table 3.3.1-1 "Reactor Protection System Instrumentation" for required actions:
		- Determine that TS 3.3.1 Condition A applies
		Determine from table 3.3.1-1 function 8 directs that Condition K applies
		- Determine form TS 3.3.1 Condition K that the bistable must be placed in trip within 72
	SRO	Direct actions to trip bistable LC-460A (LOOP 2) Hi Level Trip in the White Foxboro Rack A-12
		CUE: IF the SM is asked if Bistables should be tripped, direct the team to trip the appropriate bistables.
	ВОР	Place bistable LC-460A (LOOP 2) Hi Level Trip in the White Foxboro Rack A-12 to TRIP

### SIMULATOR OPERATOR:

Activate Trigger 2 to start the next event (22 SG Tube Leak) when directed by the lead evaluator. (Wait until PRZR level is stable from previous event, else team will have to wait in AOP-SG-1 to determine leak rate)

Scenario No.: 1

Event No.: 3

Page 1 of 3

Event Description: 22 SG Tube Leak (5 GPM)

Time	Position	Applicant's Actions or Behavior	
	ВОР	Acknowledge alarms and perform ARPs	
		- SA-1 (3-7) R49 Steam Generator Blowdown Hi Rad/Trouble (WARN SETPOINT(after about 1 minute))	
		- SA-1 (3-9) R-45 Air Ejector Radiogas Hi Rad/Trouble (after about 6 minutes)	
	TEAM	Diagnose steam generator tube leak	
	SRO	Enter 2-AOP-SG-1, "Steam Generator Tube Leak"	
	SRO/RO	Verify that Pressurizer level is being maintained by charging flow:	
		Checks PRZR level able to be maintained with two charging pumps	
		- Checks PRZR level > 11%	
		- Checks RCS Pressure and Subcooling	
	SRO	Directs RO to evaluate plant conditions for indication of Gross Tube Leakage	
		EVALUATOR NOTE: The team will not have clear indication of Gross Tube Leakage at this point.	
	TEAM	Initiate determination of affected SG. Checks:	
		- N-16 Monitor	
		- Steam Line surveys	
		- Chemistry results	
		BOOTH OPERATOR/COMMUNICATOR: When chemistry or the NPO are contacted, report back that 22 SG is affected.	
	RO/BOP	Check R-45 operable	

Scenario No.: 1

Event No.: 3

Page 2 of 3

Event Description: 22 SG Tube Leak (5 GPM)

Time	Position	Applicant's Actions or Behavior	
	ВОР	Initiate attachment 1, "R-45 Estimated Leak Rate"	
		NOTE: If chemistry is requested, provide the following information:	
		- Condenser air in leakage = 6 SCFM	
		- RCS total gaseous activity = 2.5 e-3 uCi/cc	
	ВОР	Calculates leak rate of approximately 5 gpm	
	SRO	Directs Chemistry to perform Primary to Secondary Leak Rate determination	
		BOOTH OPERATOR/COMMUNICATOR: After about 10 minutes, report that Pri to Sec leakrate is 4.3 GPM	
	SRO	Notify Health Physics of SGTL	
	BOP	Initiate Attachment 2, SG Leak Rate Data Sheet	
	TEAM	Determine primary to secondary leak rate using	
		- N-16	
		- Chemistry grab sample results	
		- Attachment 1	
		EVALUATOR NOTE: N-16 data is not valid below 30% power. Team should use Chemistry results and Attachment 1 calculation to determine that leak rate is 3-5 gpm	
	SRO	Determine that leak rate is >100 gpd	
	SRO	Direct chemistry to draw backup samples and re-perform Primary to Secondary Leak Rate calculation.	
		BOOTH OPERATOR/COMMUNICATOR: When Chemistry is contacted, report that additional samples have been obtained and that the leak rate has been verified at 4.3 gpm	

Appendix D	Required Operator Actions	Form ES-D-2

Page 3 of 3 Op-Test No.: 1 Scenario No.: 1 Event No.: 3 Event Description: 22 SG Tube Leak (5 GPM)

Time	Position	Applicant's Actions or Behavior	
	SRO	Initiate notifications per SAO-124, Oral Reporting of Non- Emergency Events and Items of Interest and Significant Occurrence Reporting	
	SRO	Direct Chemistry to calculate leak rate every 2 hours while shutting down	
	SRO	Evaluate attachment 3 and Technical Specification 3.4.13 to determine Shutdown Requirements:	
		- TS 3.4.13 Condition A – reduce leak rate to allowable in 4 hours (cannot be met)	
		- Be in mode 3 in additional 6 hours	
		- Be in mode 5 in additional 36 hours	
	SRO	Initiate plant shutdown IAW POP-3.1, "Plant Shutdown Mode 1 to Mode 3."	

# SIMULATOR OPERATOR:

Activate Trigger 3 when directed by the Lead Evaluator to proceed to the next event (23 RCP High Vibration)

10 of 19

Op-Test No.: 1 Scenario No.: 1 Event No.: 4 Page 1 of 1

Event Description: 23 RCP High Vibration

Time	Position	Applicant's Actions or Behavior
	ВОР	Acknowledge alarm and perform ARP:
		- SF 4-6 23 RCP HI Vibration
	SRO	Enter 2-AOP-RCP-1, Reactor Coolant Pump Malfunction
	TEAM	<ul> <li>Determine immediate reactor trip and RCP trip NOT required:</li> <li>Checks Stator winding temp ≥ 250°F AND Tave ≥ 547°F</li> <li>Stator winding temp ≥ 270°F AND Tave &lt; 547°F.</li> <li>RCS seal 1 ΔP &lt; 200 psig.</li> <li>Sustained RCP vibration &gt; 20 mils</li> <li>RCP motor bearing temp ≥ 200°F</li> <li>#1 Seal inlet temp ≥ 225°F</li> </ul>
	ВОР	Initiate data collection per Attachment 1, RCP Data Sheet
	SRO	Notify Plant Engineering
	ВОР	Check vibration <13 mils (no)
	ВОР	Check vibration spiking (no)
	ВОР	Check vibration trend increasing < 1 mil/hr (no)  Note: Vibration trend is increasing at a rate of 3 mils/hr
	SRO	Continue plant shutdown per POP-3.1

#### SIMULATOR OPERATOR:

Activate trigger 4 when directed by Lead Evaluator to proceed to the next event (21 MBFP trips)

F	orm ES-D-2

Аp	pendix D	Required Operator Actions

Op-Tes	Op-Test No.: 1 Scenario No.: 1 Event No.: 5 Page 1 of 1					
Event [	Event Description: 21 Main Boiler Feed Pump Trips					
Time	Position	Applicant's Actions or Behavior				
	RO	Diagnose trip of 21 Main Boiler Feed Pump				
	SRO	Diagnose Loss of Feedwater:				
		<ul> <li>Direct RO to perform immediate operator actions of 2- AOP-FW-1 Loss of Main Feedwater</li> </ul>				
	RO	Performed from memory:				
		- Checks any Main Boiler Feed Pump operating (no)				
		- Checks Reactor power greater than 4% (yes)				
		- Manually trip the reactor				
	SRO	Directs team to perform immediate operator actions of EOP E-0, Reactor Trip or Safety Injection				

## NOTE:

- 1. The SGTL will increase in size to a SG Tube Rupture (600 gpm) when the reactor is tripped (from conditional trigger 30 which actuates from condition of reactor trip breakers open)
- 2. A loss of offsite power will occur when SI is manually actuated (from conditional trigger 29 which actuates from condition of manual SI buttons pushed)

Op-Test No.: 1 Scenario No.: 1 Event No.: 6 Page 1 of 5

Event Description: Steam Generator Tube Rupture with subsequent Loss of Offsite

Power

Time	Position	Applicant's Actions or Behavior
,	SRO	Direct Operators to perform immediate operator actions of EOP-E-0, Reactor Trip or Safety Injection
	RO	Verifies Reactor Trip (from memory)
	RO	Verifies Turbine Trip (from memory)
	ВОР	Checks status of 480V ac buses (from memory)
		NOTE: Following SI actuation, offsite power will be lost and all 480V buses will be powered from the EDGs
	RO	Checks status of SI: (from memory)
		- Determines pressurizer pressure and level are lowering
		- Manually actuates SI
		- Checks both trains of SI actuated
		<b>EVALUATOR NOTE:</b> The team may go to ES-0.1 and then manually actuate SI from the foldout. Also, the team may use AOP-SG-1 IAAT step 4.1 to actuate SI.
	TEAM	Perform immediate actions steps again (using the procedure)
	SRO	Directs BOP to perform E-O Attachment 1, Automatic Action Verification
		<b>EVALUATOR NOTE:</b> See required operator actions for event 7 for BOP E-O Attachment 1, Automatic Action Verification
	RO	Verify AFW Pumps Running
	RO	Verify total AFW flow > 400 GPM

Scenario No.: 1

Event No.: 6

Page 2 of 5

Event Description: Steam Generator Tube Rupture with subsequent Loss of Offsite

Power

Time	Position	Applicant's Actions or Behavior
	RO	Verify SI System Flow
		- Checks RCS Pressure < 1160 psig (no)
		- Places one RHR pump in pullout
	RO	Check RCP Seal Cooling
		<ul> <li>Dispatch an NPO to align backup cooling to the Charging Pumps, RHR pumps, and SI pumps</li> </ul>
		Dispatch an NPO to locally close SWN-4 and SWN-5 in the zurn strainer pit
	RO	Check cold leg temperatures stable at or trending to 547°F
	RO	Check PRZR Porvs and Spray Valves Closed
	RO	Check RCPS stopped (yes- loss of offsite power)
	RO	Check if any SG is faulted (no)
	RO	Check if SG tubes are intact (no, 22 SG is Ruptured)
		- Main Steam Line radiation recorder
		- Condenser air ejector radiation recorder
		- SG Blow down radiation recorder
		- No SG Level increasing in an uncontrolled manner
	SRO	Transitions to EOP E-3, Steam Generator Tube Rupture

Op-Test No.: 1 Scenario No.: 1 Event No.: 6 Page 3 of 5 Event Description: Steam Generator Tube Rupture with subsequent Loss of Offsite Power Time Position Applicant's Actions or Behavior Critical Task Isolate feedwater flow into and steam flow from the ruptured SG before a transition to ECA-3.1 occurs E-3 -- A RO Isolate flow from rupture SG Adjust 22 SG Atmospheric Steam Dump controller to 74% Check atmospheric Steam Dump closed Trip 22 AFW Pump Dispatch NPO to locally close MS-41 Steam Supply from 22 SG to 22 AFW Pump Turbine Verify SG Blowdown Isolation Valves closed Dispatch NPO to close steam traps upstream of 22 MSIV and verify that MS-55B, 22 MSIV Bypass is closed Close MS-1-22, SS SG MSIV When 22 SG NR Level >10%, STOP Feed flow to 22 SG RO Check 22 SG pressure > 440 PSIG SRO Determine required core exit temperature: NOTE: 510°F expected with 22 SG pressure > 1025 PSIG Critical Task Establish and maintain RCS temperature so that transition from E-3 does not occur due to either of the following: E-3 -- B RCS temperature TOO HIGH to maintain required subcooling in E-3 step 16 table, OR RCS temperature TOO LOW results in a severe challenge to the Subcriticality or Integrity CSF. RO Dump steam at maximum rate from intact (21, 23, 24) SGs using atmospheric steam dumps When core exit TCs < required value (510°F), stops dumping steam Maintains core exit thermocouples less than required temperature

Op-Test No.: 1 Scenario No.: 1 Event No.: 6 Page 4 of 5 Event Description: Steam Generator Tube Rupture with subsequent Loss of Offsite Time Applicant's Actions or Behavior Position **BOP** Check Intact SG Levels Maintain total feed flow > 400 gpm until narrow range level >10% in at least one SG Check PORVs and Block Valves RO BOP Reset SI and CI Phase A **BOP** Check RCS pressure greater than 320 PSIG and then stop RHR pumps and place in AUTO RO Establish maximum charging flow RO Check 22 SG pressure stable or increasing RO Check RCS subcooling > value from table (>39°F expected) RO Depressurize RCS using PORV Opens one PORV Monitors RCS Pressure, 22 SG Pressure, PRZR Level, and RCS subcooling Closes PORV when RCS pressure < 22 SG Pressure, or PRZR Level > 71%, or subcooling < value from table

Checks RCS pressure increasing

Checks if SI flow can be terminated

AFW flow > 400 GPM

PRZR Level > 14%

RCS Pressure stable or increasing

Checks subcooling greater than value from table

RO

**TEAM** 

	Description of Occupation Astronomy	
Appendix D	Required Operator Actions	Form ES-D-2

Op-Tes	st No.: 1 So	enario No.: 1 Event No.: 6 Page 4 of 5			
Event [	Event Description: Steam Generator Tube Rupture with subsequent Loss of Offsite Power				
Time	Position	Applicant's Actions or Behavior			
	Critical Task E-3 D	Stop all SI pumps before water relief from the ruptured SG atmospheric or safety relief valves occurs			
	TEAM	STOP SI Pumps and Place in AUTO			
	TEAM	Verify SI system flow not required  - Checks Subcooling greater than value from table  - PRZR Level > 14%			

### **EVALUATOR NOTE:**

Terminate scenario after SI dumps stopped, or at the discretion of the lead evaluator.

Op-Test No.: 1 Page 1 of 1 Scenario No.: 1 Event No.: 7 Event Description: 21 SI Pump Fails to Auto Start (E-0, Attachment 1 Automatic Action Verification)

Time	Position	Applicant's Actions or Behavior
	ВОР	Verify charging system operation
		- Starts one charging pump in manual at maximum speed
		<ul> <li>Align charging system to the RWST (opens LCV-112B, Closes LC-112C, place Makeup Control Switch to STOP)</li> </ul>
	ВОР	Check 345 KV MO Disc Switch F7-9 Open
		- verifes BKR 7 and 9 open
	ВОР	Check status of 480 V buses
		- Determines all are powered from EDGs
		- Direct personnel to align lighting to TSC bus
	ВОР	Verify FW Isolation
	ВОР	Check if MSIVs should be isolated
	ВОР	Check SW system operation
	ВОР	Check Three SI Pumps running
		- Manually starts 21 SI pump
		- OPEN 851B
		- Check RHR pumps running
	ВОР	Check SI system valve aligment
	ВОР	Verify Containment Fan Coolers in service
	ВОР	Verify AFW Flow to all SGs
		NOTE: The team may use prudent operator action to isolate AFW flow to 22 SG when NR Level is > 10%
	ВОР	Verify Containment Ventilation Isolation
	ВОР	Verify Containment Isolation Phase A
	ВОР	Check if CS should be actuated
	ВОР	Verify CCR Air Conditioner status
	ВОР	Notify SRO Attachment 1 complete

# **Shift Turnover**

## **Watch Team Turnover Sheet:**

Date/Time:

**TODAY** 

Condition:

Power Ops

**RCS** Temp:

549 °F

% Power:

3%

**RCS Press:** 

2235 psig

**MW Gross:** 

0

PZR Level:

38 %

River Water:

63 °F

**RCS Total Leakage:** 

0.1 gpm

**Boron Conc:** 

1530 ppm

**RCS** Unidentified

0.1 gpm

**Control Rods** 

105 CBD

Leakage:

Xenon:

Increasing

Condenser Air leakage

6 SCFM

**EFPD:** 

30

**RCS** Gas activity

2.5E-3 μCi/cc

**PZR Press Control:** 

Channel 1

**PZR Level Control:** 

Channel 2

**Service Water:** 

3 Header Ops

**Risk Assessment:** 

Green

Daily Risk Factor:

0.78

## **Plant Equipment Status:**

No equipment out of service

#### **Instructions to the Shift:**

Continue plant startup IAW POP-1.3. Unit 2 is at 3% power, recovering from a 7 day forced outage to repair body to bonnet leak on PRZR Spray Loop 23 Bypass Valve 524. Shift orders are to continue the startup in accordance with Pop 1.3 Plant Startup, Mode 2 to Mode 1. The previous shift completed POP 1.3 though step 4.23.

Operations Manager has directed you to baseload the HP Steam Dumps to approximately 8% prior to placing the unit on line per POP-2.1

The Operations Manager, Reactor Engineering and Power Marketing have authorized a rate of power increase of 200 MWe per hour to 100% RTP.

Appendix D	Scenario Outline	Form ES-D-1

Facility: Indian Point 2	Scenario No.: NRC#2	Op-Test No.: 1
Examiners:	Operators:	

Initial Conditions: 100% Rated Thermal Power, MOL. 21 EDG is out of Service. 22 Charging Pump is out of service.

Turnover: Unit 2 is at 100% Power steady state conditions 340 EFPD. 21 EDG is out of service and has been inoperable for 42 hours. Maintenance is currently performing repairs.

In addition, 22 Charging Pump was removed from service for corrective maintenance 18 hours ago. Expected return to service in 35 hours.

ļ			
Event No.	Malf. No.	Event Type*	Event Description
1	RLY- DSG009	N BOP/SRO	23 EDG inoperable due to 86 Lockout Relay tripped Begin TS required plant shutdown
٠١		R RO	
2	MAL- EPS007D	C ALL	Loss of Bus 6A (Lose 23 CHG Pump, Starts 21 CHG Pump)
3	MOT-	C	21 Charging Pump Trips
	CVC003A	ALL	Manual Reactor Trip
4	MAL- EPS001	M ALL	Loss of all AC Station Aux Xfmr fails
	MAL- DSG003B		22 EDG fails to start
5	MOC- SWS010	C SRO/BOP	25 SW Pump Fails to auto start following start of 22 EDG and energizing associated 480V buses
	MOC- SWS-011		

<sup>\* (</sup>N)ormal (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

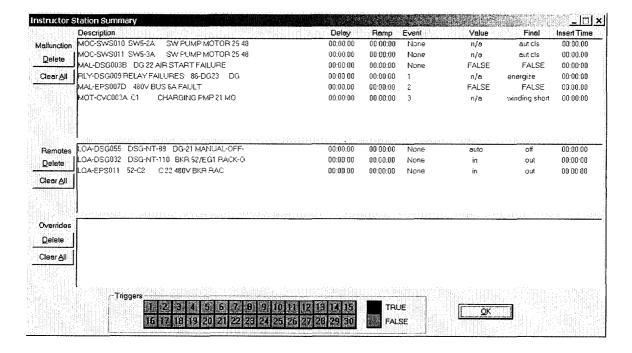
1. Execute batch file Bat "NRC2.bat from Ph.D Expert window:

^LOA-DSG055 DSG-NT-89 DG-21 MANUAL-OFF-AUTO (ECS) IRF LOA-DSG055 (-1 0) OFF ^ LOA-DSG032 DSG-NT-110 BKR 52/EG1 RACK-OUT/RACK-IN IRF LOA-DSG032 (-1 0) OUT ^LOA-EPS010 52-C1 C 21 480V BKR RACK OUT/ RACK IN IRF LOA-EPS010 (-1 0) OUT ^ MOC-SWS010 SW5-2A SW PUMP MOTOR 25 480V BUS 2A IMF MOC-SWS010 (-1 0) 4 ^ MOC-SWS011 SW5-3A SW PUMP MOTOR 25 480V BUS NO. 3A IMF MOC-SWS011 (-1 0) 4 ^ MAL-DSG003B DG 22 AIR START FAILURE IMF MAL-DSG003B (-1 0) TRUE ^ RLY-DSG009 RELAY FAILURES: 86-DG23 DG23 LOCKOUT RELAY IMF RLY-DSG009 (1 0) 3 ^ MAL-EPS007D 480V BUS 6A FAULT IMF MAL-EPS007D (20) TRUE ^ MOT-CVC003A C1 CHARGING PMP 21 MOTOR IMF MOT-CVC003A (3 0) 3 ^ MOV 625 fails closed after 204 closed TRGSET 30 "xcoi213o.eq.0" TRG 30 "IMF MOV-CCW008 (-1 0) 2 " ^ Loss of Statin Aux Transforem 60 seconds after RX Trip TRGSET 29 "jbkrta.eq.0" TRG 29 "IMF MAL-EPS001 (-1 60) TRUE"

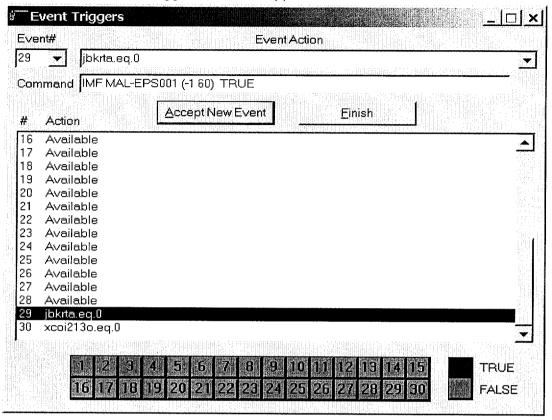
- 2. Place CS for 21 EDG in Pullout
- 3. Place 23 Charging pump in auto operation.
- 4. Place CS for 22 Charging Pump in Pullout
- 5. Verify that standard PICs screens are displayed (no screens that could cue the team to upcoming events)

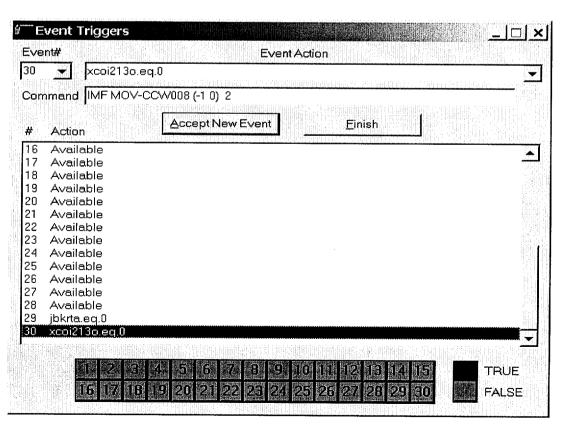
#### Validation time 105 minutes

Verify the following commands appear in the instructor station summary:



Verify that conditional trigger 29 and 30 appear as follows:





Scenario No.: 2

Event No.: 1

Page 5 of 2

Event Description: 23 EDG becomes Inoperable due to 86 Lockout Tripped. Begin TS

required plant shutdown

# SIMULATOR OPERATOR:

Activate Trigger 1 to insert the 23 EDG 86 Relay Trip when directed by the lead evaluator

Time	Position	Applicant's Actions or Behavior
	ВОР	Acknowledge alarm and perform Alarm Response Procedure:
		- SG 4-9 23 Diesel Generator Trouble
		- Dispatch NPO to 23 EDG
		- Refer to SOP 27.3.1.3 23 Emergency Diesel Generator Manual Operation
		SIMULATOR OPERATOR/COMMUNICATOR:
		After dispatched, NPO reports back to CCR that 23 EDG 86 lockout has tripped.
	SRO/BOP	Direct NPO to attempt to clear trouble and VERIFY that lockout relay is RESET
		SIMULATOR OPERATOR/COMMUNICATOR:
		NPO reports back to CCR that 23 EDG 86 Lockout relay will NOT reset.
	SRO	Refers to Technical Specification 3.8.1 Condition E
		- Two or more EDGs inoperable
		<ul> <li>Action Statement E.1 Restore at least two EDGs to operable status within 2 hours</li> </ul>
		<ul> <li>Determines that plant shutdown must be commenced per action statement F.1, be in mode 3 in 6 hours</li> </ul>
		SIMULATOR OPERATOR/COMMUNICATOR:
		Maintenance reports that 21 and 23 EDG cannot be returned to service within 2 hours. The 86 tripped and will not reset. It appears that the relay itself must be replaced.
		Note: If the team does not begin the SD right away, then the Operations Manger directs the team to commence shutdown immediately.
	SRO	Directs plant shutdown per POP-3.1, Plant Shutdown, Mode 1 to Mode 3

Appendix D	Required Operator Actions	Form ES-D-2

Op-Tes	st No.: 1	Scenario No.: 2 Event No.: 1 Page 2 of 2
Event Description:		23 EDG becomes Inoperable due to 86 Lockout Tripped. Begin TS required plant shutdown
Time	Position	Applicant's Actions or Behavior
	RO	Adds negative reactivity:  - Boration  - Control rod insertion  - Maintains Delta-flux within the target band
	ВОР	Lowers turbine load using Turbine Governor (or LL1/LL2 if directed)

## SIMULATOR OPERATOR:

Activate Trigger 2 to insert the Loss of Bus 6A when directed by the lead evaluator

Scenario No.: 2

Event No.: 2

Page 7 of 2

Event Description: Loss of Bus 6A (Lose 23 Charging Pump)

Time	Position	Applicant's Actions or Behavior
	ВОР	Diagnose loss of Bus 6A. Acknowledge multiple alarms:
		- SH 3-10 480V Swgr Motor Trip (Common)
		- SH 4-10 480V Bus Main Tie or Emerg Gen Breaker Trip
		- SG 4-6 Switch Gear 21 or 22 Undervoltage
		<ul> <li>Additional alarms associated with loss of 26 SWP, 23         Charging Pump, 22 Rod Drive MG, 25 FCU, Process         Radiation Monitors, 22 PW Pump, 22 BA Xfer Pump     </li> </ul>
	SRO	Enters 2-AOP-480V-1, Loss of Normal Power to Any 480V Bus and directs team actions
	RO	Starts 21 Charging Pump
	ВОР	Checks one CCW pump running (yes)
		Checks OPEN MOV-625 RCP Thermal Barrier Discharge Isol VIv (yes)
		Checks SW pumps running on essential and non-essential headers (yes)
	RO	Starts 21 PW Pump
	SRO	Dispatches NPO to monitor all running EDGs
		NOTE: 22 EDG fails to start. NPO is unable to start it if directed to. 21 and 23 EDGs are inoperable. TS 3.8.1 Condition E still applies for two or more EDGs inoperable. NPO report 22 EDG OVERCRANK Alarm is annunciated.

Appendix D	Required Operator Actions	Form ES-D-2

Op-Tes	t No.: 1 Se	cenario No.: 2 Event No.: 2 Page 2 of 2						
Event Description: Loss of Bus 6A (Lose 23 Charging Pump)								
Time	Position	Applicant's Actions or Behavior						
	ВОР	Performs Attachment 3 Restoring Normal Power to 480V Bus 6A:						
		- Dispatch NPO to start 23 EDG						
		- Notifies SRO that 23 EDG cannot be started						
		<ul> <li>Diagnoses fault on bus 6A (observes 6A fault lamp extinguished)</li> </ul>						
		<ul> <li>Dispatch NPO to open bus 6A MCC breakers and perform visual inspection of Bus 6A</li> </ul>						
		- Places Control Switches for Bus 6A loads to pullout:						
		SIMULATOR OPERATOR/COMMUNICATOR:						
		Report that the back of Bus 6A switchgear appears damaged and smells burned.						
	SRO	Refer to attachment 5 "480V Bus Equipment" for loads lost and perform a review of Technical Specifications for required actions - Refers to TS 3.8.9 Condition A						

### SIMULATOR OPERATOR:

Activate Trigger 3 to insert Trip of 21 Charging Pump when directed by the lead evaluator

Scenario No.: 2

Event No.: 3

Page 9 of 4

Event Description: 21 Charging Pump Trips (Loss of all charging)

Time	Position	Applicant's Actions or Behavior
	RO	Diagnose trip of 21 Charging Pump
	SRO	Enter 2-AOP-CVCS-1, Chemical and Volume Control System Malfunction and Directs operator actions
	RO	Closes the following valves
		- 200A Letdown Orifice 75 GPM
		- 200B Letdown Orifice 45 GPM
		- 200C Letdown Orifice 75 GPM
		- LCV-459 Letdown Stop
	ВОР	Closes 204A Alternate Charging 22 Hot Leg
000		NOTE: MOV-625 should close at this time
	ВОР	Observe MOV-625 RCP Thermal Barrier Disc CLOSED
		- Place control switch for MOV-625 in OPEN positon
		- Determine MOV 625 does not open
	SRO	Directs team to
		- Trip Reactor
		- Stop all RCPs
		- Go to E-0
	RO	Actuates manual reactor trip
	ВОР	Stops all RCPs
	SRO	Directs team to perform immediate actions of E-0

Scenario No.: 2

Event No.: 4

Page 2 of 4

Event Description: Loss of All AC Power

NOTE: Loss of all AC occurs 60 seconds after reactor is tripped

Time	Position	Applicant's Actions or Behavior
	SRO	Directs team actions in EOPs
	RO	Verifies Reactor Trip
		Verifies Turbine Trip
	ВОР	Checks status of AC Buses
		- Determines that no AC bus is energized
	SRO	Enters EOP ECA-0.0 Loss of All AC Power
		- Directs team to perform ECA-0.0 immediate actions
	RO	Verifies Reactor Trip
		Verifies Turbine Trip
	RO	Verifies RCS isolated. Observes the following valves CLOSED:
		- PRZR PORVS,
		- LCV-459, Letdown Stop
		- 200A, B, C, Letdown Isolation valves
		- 213, Excess letdown isolation
	Critical Task ECA-0.0B	Establish at least 400 gpm AFW flow to the SGs before SG dry-out occurs (WR level less than 14%)
	ВОР	Align AFW to SGs using 22 Turbine Drive AFW Pump
		- Checks Open PCV-1139, steam supply regulator
		- Manually aligns TD AFW Flow Control Valves
		- Adjust HCV-1118, steam supply
		- Observes > 400 GPM flow

Op-Tes	t No.: 1 So	cenario No.: 2 Event No.: 4 Page 3 of 4					
Event D	Event Description: Loss of All AC Power						
Time	Position	Applicant's Actions or Behavior					
	TEAM	Attempt to restore Power to any 480V Bus					
		Dispatch NPO to emergency start EDGs					
		SIMULATOR OPERATOR:					
		NPO is initially unsuccessful at starting an EDG.					
	ВОР	Performs 2-AOP-138KV-1, Loss of Power to 6.9KV Bus 5 AND/OR 6					
		SIMULATOR OPERATOR:					
		CONED DO provides permission use Feeder 13W92. However, 22EDG will be started by NPO BEFORE bus 5A is restored.					
	RO	Places Control Switches- in Pull-Out:  - CS Pumps - SI Pumps - FCUs - MD AFW Pumps - Turning Gear Oil Pump - Bearing Oil Pump - Turb Aux Oil Pump - CCW Pumps - RHR Pumps					
	SRO	Dispatch personnel to locally restore AC power					
		SIMULATOR OPERATOR/COMMUNICATOR:					
		After this step is performed, report back to CCR that 22 EDG is ready to start. When permission is given to start 22 EDG, then delete the air start malfunction MAL-DSG003B and execute batch file bat EDG22-1.bat to reset and start 22 EDG					
		Report back immediately after 22 EDG starts.					

Op-Tes	st No.: 1 So	cenario No.: 2 Event No.: 4 Page 4 of 4							
Event [	Description: Lo	ss of All AC Power							
Time	Position	on Applicant's Actions or Behavior							
	Critical Task ECA-0.0F	Manually start the ESW pump such that the EDG does not fail because of damage caused by engine overheating							
	вор	Observe 22 EDG has started and bus 2A/3A energized - Starts 25 Service Water Pump							
		- Checks 1276 and 1276 A Diesel Generator Cooling Water valves open							
	TEAM	Verify equipment loaded onto energized bus:							
		- MCC 24							
		- MCC 24A							
ı		- 22 Battery Charger							
		- 22 Static Inverter on alternate power							
		- PA system inverter							
		- MCC 26C							
		- MCC 211							
16		- 23 Battery Charger							
	SRO	Selects Recovery Procedure based upon conditions							
		- ECA-0.1, Loss of All AC Power Recovery Without SI Required (expected)							
		- ECA-0.2, Loss of All AC Power Recovery With SI Required (if RCP seals degrade)							

## NOTE:

Terminate the scenario when the transition from ECA-0.0 is made, or at the discretion of the lead examiner.

## **Shift Turnover**

### Watch Team Turnover Sheet:

Date/Time:

**TODAY** 

**Condition:** 

Power Ops

**RCS Temp:** 

562 °F

% Power:

100%

**RCS Press:** 

2235 psig

**MW Gross:** 

1017

**PZR Level:** 

48 %

River Water:

78 °F

**RCS Total Leakage:** 

0.1 gpm

**Boron Conc:** 

972 ppm

**RCS** Unidentified

0.1 gpm

**Control Rods** 

214 CBD

Leakage:

Xenon:

Equilibrium

Condenser Air leakage

6 SCFM

**EFPD:** 

340

**RCS** Gas activity

2.54e-2 μCi/cc

**PZR Press Control:** 

Channel 1

**PZR Level Control:** 

Channel 2

Service Water:

3 Header Ops

**Risk Assessment:** 

Yellow

Daily Risk Factor:

3.45

#### **Plant Equipment Status:**

- 1. EDG 21 is out of service for bearing replacement. It was removed from service 42 hours ago and is due back in 12 hours. Maintenance is currently performing repairs. TS 3.8.1.B surveillance (2ptw19) requirement last performed 3 hours ago.
- 2. 21 and 22 EDGs are protected equipment.
- 3. 22 Charging Pump was removed from service for corrective maintenance 18 hours ago. Expected return to service in 35 hours.

#### **Instructions to the Shift:**

Maintain current plant conditions.

Appendix D	Scenario Outline	Form ES-D-1
Appendix D	Scenario Outiline	1 01111 E3-D-1

Facility: Indian Poin	t 2 Scenario No.	.: NRC#3	Op-Test No.: 1
Examiners:		Operators:	
		<del></del>	

Initial Conditions: 40% Rated Thermal Power, MOL

Turnover: Unit 2 is at 38% Power with power ascension to 100% in progress following a forced outage. No equipment OOS.

Event No.	Malf. No.	Event Type*	Event Description
1		N SRO/BOP R RO	Raise reactor power
2	MAL- CRF001AY	C ALL	Stuck Control Rod (P-10)
3	XMT- SGN026A	I RO/SRO	LT 447B 24 SG Controlling Level Channel fails low.
4	MAL- CCW001D	C BOP/SRO	RCP Upper Bearing Oil Cooler CCW leak
5	MAL- RCS001C	M ALL	LBLOCA
6	RLY- PPL487 RLY- PPL488	C RO	Safety Injection auto actuation failure, RO manually actuates
7	MOC- RHR001 MOC- RHR002	C ALL	Both Recirculation Pumps both fail to start

<sup>\* (</sup>N)ormal (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

#### Reset to IC-125

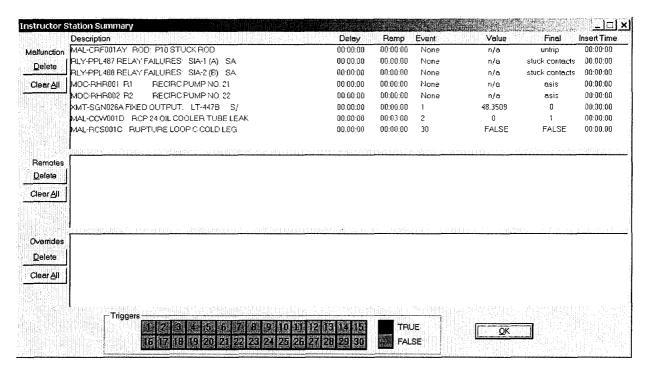
Execute batch file Bat "NRC3.bat from Ph.D Expert window:

```
^ MAL-CRF001AY ROD: P10 STUCK ROD
IMF MAL-CRF001AY (-1 0) 1
^ RLY-PPL487 RELAY FAILURES: SIA-1 (A) SAFETY INJECTION AUTO MASTE
IMF RLY-PPL487 (-1 0) 2
^ RLY-PPL488 RELAY FAILURES: SIA-2 (B) SAFETY INJECTION AUTO MASTE
IMF RLY-PPL488 (-1 0) 2
                     RECIRC PUMP NO. 21 MOTOR
^ MOC-RHR001 R1
IMF MOC-RHR001 (-1 0) 5
                      RECIRC PUMP NO. 22 MOTOR
^ MOC-RHR002 R2
IMF MOC-RHR002 (-1 0) 5
^ XMT-SGN026A FIXED OUTPUT: LT-447B
                                        S/G 24 LEVEL
IMF XMT-SGN026A (1 0) 0.000000 0 48.796101
^ MAL-CCW001D RCP 24 OIL COOLER TUBE LEAK
                                               (NR)
IMF MAL-CCW001D (2 0) 1.000000 180 0.000000
^ MAL-RCS001C RUPTURE LOOP C COLD LEG
                                              (NR)
IMF MAL-RCS001C (30 0) TRUE
```

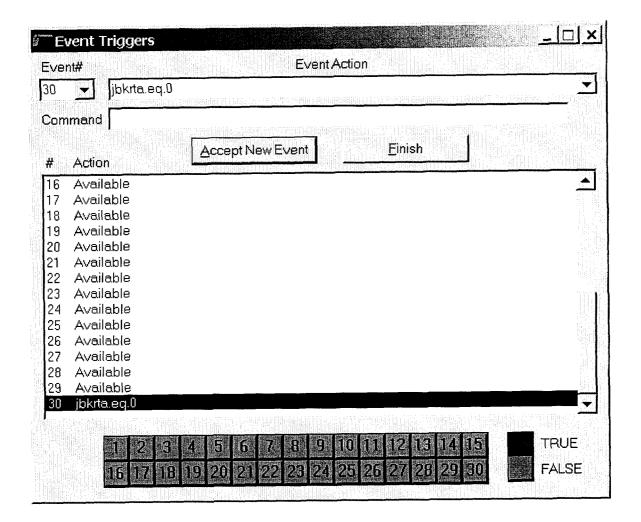
Verify the following commands appear in the instructor station summary:

^ Set Up trigger to actuate LBLOCA when Rx Trips

TRGSET 30 "jbkrta.eq.0"



Verify that conditional trigger 30 appears as follows



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Required Operator Actions

Op-Test No.: 1 Scenario No.: 3 Event No.: 1 Page 1 of 1					
Event Description: Raise reactor power					
Time	Position Applicant's Actions or Behavior				or Behavior
	SRO	Directs activities associated with power increase per 2-POP-1.3, "Plant Startup, Mode 2 to Mode 1":			
		- Rod Motion			
		- Dilution			
		- Tu	bine	e Load	
	RO	Adds Positive reactivity - Rod Motion			
		- Dilution			
	ВОР	Raises turbine load using Turbine Governor (or LL1/LL2 if directed)			
		1		rator Flux surveys have tion steam Bypass va	•
				conditioning 3% per ho quired.	ur limit on power increase is
				stioned about the MBF investigating the low o	P seal water injection pump lelta-P alarm setpoint.

## **EVALUATOR NOTE:**

Proceed to the next event after sufficient rod motion has resulted in positive indication of rod misalignment

Op-Test No.: 1 Scenario No.: 3 Event No.: 2 Page 1 of 1

Event Description: Stuck Control Rod (P-10)

Timo	Position	Applicant's Actions or Pobavior	
Time	Position	Applicant's Actions or Behavior	
	RO	Diagnoses Control Rod misalignment	
	вор	Acknowledges alarm and performs ARP: - SF 2-7 Control Rod or Power Distribution Trouble	
	SRO	Enters 2-AOP-ROD-1, Rod Control and Indication Systems Malfuncations - Directs the actions of the ROs	
	RO	Determines Control Rod P-10 is misaligned by evaluating the following indications: - Flight Panel DVM - PICs	
	SRO	Notifies Reactor Engineering of control rod misalignment.  NOTE: If reactor engineering is contacted to determine if P-10 is misaligned, then reactor engineering responds that P-10 is	
		misaligned from its group.	
	RO	Places rod control in MANUAL	
	ВОР	Adjusts turbine load as necessary to maintain Tave on program	
	SRO	Evaluates the following Technical Specifications: - 3.1.4 Rod Group Alignment Limits	
	RO	Determines actual position of rod P-10  EVALUATOR NOTE:  If I&C assistance is requested to determine rod position using RPI drawer, give the following cue: RPI drawer 2.60 volts.	

# SIMULATOR OPERATOR:

Activate Trigger 2 to fail LT-447B when directed by the lead examiner

Op-Test No.: 1 Scenario No.: 3 Event No.: 3 Page 1 of 1 Event Description: LT 447B, 24 SG Controlling Level Channel fails low.

Time	Position	Applicant's Actions or Behavior	
	RO	Diagnoses 24 SG Level control deviation	
	ВОР	Performs ARP for associated alarms:	
		- FB 1-1 Steam Generator Level Control Deviation	
		- SB-2 4-3 24 SG Low Level Channel Trip	
	SRO	Directs RO to perform immediate action of 2-AOP-INST-1, Instrument/Controller Failures	
	RO	Diagnoses failure of LT-447B, 24 SG controlling level channel	
		<ul> <li>Places 24 SG Feedwater Regulating Valve controller in Manual and adjusts FRV position as necessary to control 24 SG level at program.</li> </ul>	
	SRO	Evaluates the following Technical Specifications:	
		- 3.3.1 Table 1 Function 13 (condition E applies)	
*		- 3.3.2 Table 1 Functions 5.b and 6.b (condition D applies)	
		- Determines that bistable must be tripped within 72 hours per 3.3.1 Condition E and 3.3.2 Condition D	
	ВОР	Places bistable trip switches in trip:	
		- LC-447E, Loop 4B High Level, in Foxboro Blue Rack B-2	
		- LC-447F, Loop 4B Low Level, in Foxboro Blue Rack B-2	

<sup>\*</sup> EXAMINER NOTE: the next event takes ~6 minutes until the first alarm providing a cue to the operator. Therefore, the next malfunction should be activated accordingly.

#### SIMULATOR OPERATOR:

Insert the next malfunction by activating trigger 2 to cause RCP Upper Bearing Oil Cooler CCW leak when directed by the lead evaluator.

Page 1 of 1

Event No.: 4

Scenario No.: 3

Op-Test No.: 1

Event D	escription: R	CP Upper Bearing Oil Cooler CCW leak
Time	Position	Applicant's Actions or Behavior
	ВОР	Acknowledge alarm and perform ARP:  - SA 3-1, Reactor Coolant Pump Motor Bearing Oil High Level 1.25"  - Observes status light and identifies 24 RCP Upper bearing High lamp illuminated  - Monitors motor bearing temperatures and vibration
	SRO	Enters 2-AOP-RCP-1 Reactor Coolant Pump Malfunctions and directs the actions of the team
	ВОР	Monitors the following parameters:  - Stator Winding Temperature  - #1 Seal Delta-P  - RCP Vibrations  - RCP Motor Bearing Temperatures  - #1 Seal Inlet Temperatures  Notifies the SRO WHEN RCP Motor Bearing Temperatures exceed 200°F (or at a lower value provided by the SRO)  NOTE: RCP BRG Temp High Alarm at 180F
	ВОР	Monitors RCP motor bearing temperatures and notifies the SRO when 24 RCP motor bearing temperature is >= 185°F
	SRO	When 24 RCP motor bearing temperature >= 185°F, INITIATES plant shutdown using POP-3.1 (with goal of stopping 24 RCP prior to reaching 200°F)
	SRO	When 24 RCP temperatures reach 200°F (or earlier if a band is provided by the SRO) directs the team to trip the Reactor, Stop 24 RCP and INITIATE EOP E-0
	RO	Trips the reactor
	ВОР	Stops 24 RCP
	SRO	Directs team to perform immediate actions of EOP E-0, Reactor Trip or Safety Injection

# **EXAMINER NOTE:**

Proceed to the next event after the reactor is tripped. The LBLOCA will occur automatically from the reactor trip.

Appendix D	Required Operator Actions	Form ES-D-2
Appendix D	riequired Operator Actions	1 01111 20-0-2

Op-Test No.: 1 Page 1 of 1 Scenario No.: 3 Event No.: 5 Event Description: LBLOCA Applicant's Actions or Behavior Time Position Directs team to perform immediate operator actions of EOP-E-0, SRO Reactor Trip or Safety Injection RO Verifies Reactor Tripped RO Verifies Turbine Tripped BOP Verifies all 480V buses energized

Op-Test No.: 1 Scenario No.: 3 Event No.: 6 Page 1 of 3 Event Description: Safety Injection auto actuation failure, RO manually actuates

Time	Position	Applicant's Actions or Behavior	
	Critical Task E-0 D:	Manually actuate at least one train of SIS-actuated safeguards before any of the following:  - Transition to any E-1 series, E-2 series, or E-3 series procedure or transition to any FRP  - Completion of step 5.a of ES-0.1	
	RO	Checks Status of Safety Injection	
		- Determines that neither train of SI is actuated	
ı.		- Manually actuates both trains of Safety Injection	
	ВОР	Performs Attachment 1 while SRO and RO continue in E-0:     Verify charging system operation     Starts one charging pump in manual at maximum speed     Align charging system to the RWST (opens LCV-112B, Closes LC-112C, place Makeup Control Switch to STOP)     Check 345 KV MO Disc Switch F7-9 Open     verifes BKR 7 and 9 open     Check status of 480 V buses     Dispatches NPO to reset all lighting and MCCs 24A, 27A, and 29A     Stops all Condensate Pumps     Verify FW Isolation     Check if MSIVs should be isolated     Check SW system operation     Check SI System Operation     Check SI system valve aligment     Verify Containment Fan Coolers in service     Verify AFW Flow to all SGs     Verify Containment Isolation Phase A     Check if CS status     Verify CCR Air Conditioner status     Notify SRO Attachment 1 complete	

9 of 14

Op-Test No.: 1 Scenario No.: 3 Event No.: 6 Page 2 of 3

Event Description: Safety Injection auto actuation failure, RO manually actuates

Time	Position	Applicant's Actions or Behavior	
	RO	Observes RCS Subcooling less than 30°F and at least one SI Pump Running	
		Stops all RCPs	
	RO	Verifies AFW Pumps running and supplying > 400 GPM AFW flow to the SGs	
		Verifies SI Pump flow and RHR pump flow	
		Checks RCP seal cooling	
		Starts 22 SWP	
		Controls RCS Cool-Down by verifying not dumping steam, reducing AFW flow to >400gpm check MSIVs closed	
		Checks PORVS and Spray Valves closed	
	SRO	Dispatches NPO to close SWN-4 and SWN-5 in the Zurn strainer pit	
	RO	Checks for Faulted and ruptured SGs	
	SRO/RO	Diagnose RCS is not intact	
	SRO	Transition to EOP E-1, Loss of Reactor or Secondary Coolant	

Op-Test No.: 1 Scenario No.: 3 Event No.: 6 Page 3 of 3 Event Description: Safety Injection auto actuation failure, RO manually actuates Time Position Applicant's Actions or Behavior RO Verifies RCPs stopped Check for any faulted SG Checks SG levels and maintains >400gpm until one SG level > 27% Opens one Block Valve **BOP** Resets SI Resets Phase A and Phase B Opens PCV-1228 to supply IA to containment RO Check for ruptured SG Check charging flow Notifies team when RWST level < 9.24 feet **BOP** SRO Transitions to ES-1.3, Transfer to Cold Leg Recirculation

Op-Test No.: 1 Scenario No.: 3 Event No.: 7

Page 1 of 2

Event Description: Both Recirculation Pumps both fail to start

Time	Position	Applicant's Actions or Behavior
	Team	Verify SI reset (Resets SI if not previously performed)
	SRO	Dispatch NPO to fully open SWN-35 and SWN-35-1 CCW Heat Exchanger SW Outlet valves
	RO	Stop all Charging Pumps
		Turn off all PRZR Heaters
		Reset Containment Spray
		Place Recirc Switches 1 and 3 to ON
	SRO	Check SW aligned for three header
		Check SWN-4 an SWN-5 have been closed
	RO	Place SI Recirc Switch 2 to ON
		Check Containment Sump Level Greater than 47'10"
		Place SI Recirc Switch 4 to ON  - Determines 21 Recirculation Pump will not start  - Attempts to manually start 22 Recirculation Pump  - Determines neither Recirculation Pump can be started
	Critical Task ES-1.3 A	Transfer to cold-leg recirculation and establish minimum required ECCS recirculation flow per ES-1.3 Step 10 Table prior to core uncovery based upon RVLIS indication (41%).
	SRO	Performs Attachment 2 Cold Leg Recirculation Using RHR Pumps
	RO	Establish Cold Leg Recirculation Using RHR Pumps - Check 885A and 885B both energized

Appendix D Required Operator Actions	Form ES-D-2
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Op-Test No.: 1 Scenario No.: 3 Event No.: 7 Page 2 of 2

Event Description: Both Recirculation Pumps both fail to start

Time	Position	Applicant's Actions or Behavior
	SRO	Dispatch NPO to energize MOV-882
	RO	Verify Recirc switches 3, 4, and 5 in OFF Verify both RHR pumps stopped Verify both Recirc pumps stopped Verify 1802A and 1802 B closed CLOSE MOV-882 Open MOV-885A and MOV-885B Start 22 RHR Pump
	SRO	Return to ES-1.3 step 10  Determine if adequate low head recirculation flow has been established
	RO	Checks flow indicators 946A-D and provides values to SRO Place SI Recirc Switch 7 to ON Place SI Recirc Switch 8 to ON

# **EXAMINER NOTE:**

Terminate the scenario after adequate Recirc flow has been established and verified, or at the Lead Examiner's discretion.

# **Shift Turnover**

# **Watch Team Turnover Sheet:**

Date/Time:

TODAY

Condition:

Power Ops

**RCS Temp:** 

552°F

% Power:

38%

**RCS Press:** 

2235 psig

**MW Gross:** 

325

**PZR Level:** 

41 %

**River Water:** 

63°F

**RCS Total Leakage:** 

0.1 gpm

**Boron Conc:** 

1138 ppm

**RCS** Unidentified

0.1 gpm

Control Rods

178 CBD

Xenon:

Leakage:

Increasing

Condenser Air leakage

6 SCFM

EFPD:

340

**RCS** Gas activity

2.54e-2 μCi/cc

**PZR Press Control:** 

Channel 1

**PZR Level Control:** 

Channel 2

**Service Water:** 

3 Header Ops

**Risk Assessment:** 

Green

Daily Risk Factor:

0.79

#### **Plant Equipment Status:**

1. The unit was forced offline four days ago. Repairs have been completed and restart is in progress.

# **Instructions to the Shift:**

- 1. Continue startup per POP-1.3. Step 4.66 is currently in progress.
- 2. OM and Reactor Engineering have approved rate of power increase to 100% at 200 MWe per hour.

Appendix D	Scenario Outline	Form ES-D-1

-Facility: Indian Point	2 Scenario No.: NRC#4	Op-Test No.: 1
Examiners:	Operators:	

Initial Conditions: 100% Rated Thermal Power, MOL. 21 EDG is out of Service. 21 Charging Pump is out of service.

Turnover: Unit 2 is at 100% Power steady state conditions 340 EFPD. 21 EDG is out of service and has been inoperable for 42 hours. Maintenance is currently performing repairs.

In addition, 22 Charging Pump was removed from service for corrective maintenance 18 hours ago. Expected return to service in 35 hours.

Event No.	Malf. No.	Event Type*	Event Description
1		N ALL	Raise main generator reactive load (MVARS)
2	MOT- CFW003A	C BOP/SRO R RO	23 Condensate Pump trips Reduce steam flow <feed and="" boration="" flow="" insertion<="" reduce="" rod="" tave="" td="" using=""></feed>
3	XMT- CVC019A	I ALL	VCT Level Transmitter fails low
4	MAL- SGN002B Bat FailRxTrips.bat	M ALL	Faulted Steam Generator Reactor auto and manual trips fail to actuate
5	AOV- RCS002A	C RO/SRO	PORV Fails Open. Block valve used to isolate it.

<sup>\* (</sup>N)ormal (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

### Simulator Setup

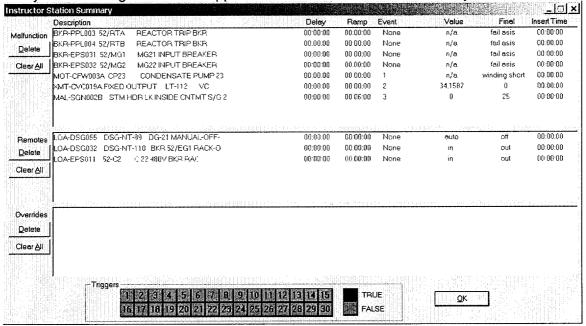
- 1. Reset to IC-2, 100% MOL
- 2. Execute batch file Bat "NRC4.bat from Ph.D Expert window:

^ BKR-PPL003 52/RTA REACTOR TRIP BKR IMF BKR-PPL003 (-1 0) 5 ^ BKR-PPL004 52/RTB REACTOR TRIP BKR IMF BKR-PPL004 (-1 0) 5 ^ BKR-EPS031 52/MG1 MG21 INPUT BREAKER (480) IMF BKR-EPS031 (-1 0) 5 ^ BKR-EPS032 52/MG2 MG22 INPUT BREAKER (480) IMF BKR-EPS032 (-1 0) 5 ^LOA-DSG055 DSG-NT-89 DG-21 MANUAL-OFF-AUTO (ECS) IRF LOA-DSG055 (-1 0) OFF ^LOA-DSG032 DSG-NT-110 BKR 52/EG1 RACK-OUT/RACK-IN IRF LOA-DSG032 (-1 0) OUT ^ LOA-EPS011 52-C2 C 22 480V BKR RACK OUT/RACK IN IRF LOA-EPS011 (-1 0) OUT ^ MOT-CFW003A CP23 CONDENSATE PUMP 23 MTR IMF MOT-CFW003A (1 0) 3 ^ XMT-CVC019A FIXED OUTPUT: LT-112 VCT LEVEL IMF XMT-CVC019A (2 0) 0.000000 0 34.159801 ^ MAL-SGN002B STM HDR LK INSIDE CNTMT S/G 22 (NR) IMF MAL-SGN002B (3 0) 25.000000 360 0.000000 ^ set up conditional trigger 30 to actuate when PORV opens TRGSET 30 "xeoo325r.eq.1" ^ Fails open PORV 455C four minutes after it auto opens TRG 30 "IMF AOV-RCS002A (-1 240) 1 "

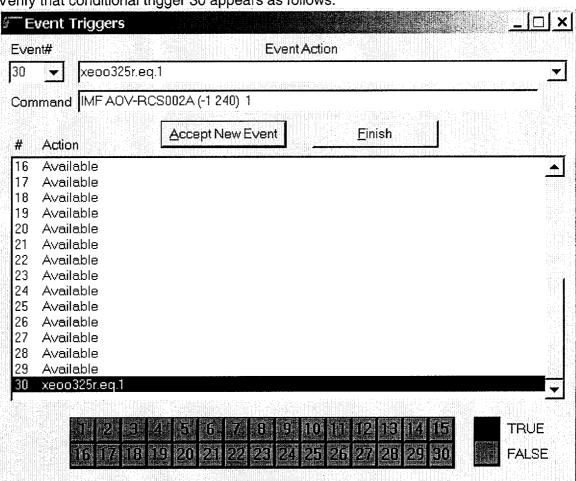
- 3. Place CS for 21 EDG in Pullout
- 4. Place 23 Charging pump in auto operation.
- 5. Place CS for 22 Charging Pump in Pullout
- 6. Put a copy of SOP 26.4 MVARs section on RO clipboard.

# Simulator Setup

Verify the following commands appear in the instructor station summary



Verify that conditional trigger 30 appears as follows:



Appendix D Required Operator Actions Form ES-D-				
	Appendix D	R	equired Operator Actions	Form ES-D-2

Op-Test No.: 1 Scenario No.: 4 Event No.: 1 Page 4 of 2

Event Description: Raise Main Generator Reactive Load (MVARS)

Simulator Operator: Telephone the CCR from the System Operator. Request Indian Point Unit 2 raise VARs 100 MVARs OUT.

Time	Position	Applicant's Actions or Behavior
	CRS	Receives VAR request from SO. (Jim Armstrong)
		Directs RO to increase lagging MVARs by 100
		Directs BOP to monitor UAT voltage during main generator voltage adjustment.
	RO	Refers to 2-SOP-26.4
	ВОР	If necessary, place Unit Aux Transformer tap changer in Manual and adjust UAT voltage to 7.0 to 7.1 KV while adjusting Main Generator Voltage.
	RO	Adjust Main Generator Voltage using the AC Raise switch to obtain desired VARS
	ВОР	When VARs adjustment is complete, verify UAT voltage 7.0 to 7.2 KV
	ВОР	Return the UAT Tap Changer to AUTO

Simulator Operator: When directed by the lead examiner, activate Trigger 1

Op-Test No.: 1 Scenario No.: 4 Event No.: 2 Page 1 of 1

Event Description: 23 Condensate Pump trips, Reduce Steam flow<Feed Flow, Reduce

Tave using boration and rod insertion

# SIMULATOR OPERATOR:

Activate Trigger 1 when directed by Lead Examiner to start event.

Time	Position	Applicant's Actions or Behavior
	ВОР	Diagnoses trip of 23 Condensate Pump
	SRO	Directs RO to perform immediate actions of 2-AOP-FW-1, Loss of Main Feedwater
	RO	Verifies Main Feedwater Pumps running
	SRO	Directs team actions using 2-AOP-FW-1, Loss of Main Feedwater
	RO	Reduces turbine load as necessary to maintain Feed Flow ≥ Steam Flow
		Adds negative reactivity using boration and/or control rods to maintain Tave on program
		Monitors delta-flux during reactivity addition and uses control rods when required to maintain delta-flux within the target band
	ВОР	Monitors MBFP suction pressure and if suction pressure cutback actuates, then places MBFP Master Speed Controller in MANUAL and slowly lowers MBFP speed to maintain suction pressure > 310 psig
		EXAMINER NOTE: Suction pressure will not immediately respond after manual action is taken: Suction pressure will respond after Feed Flow is > Steam Flow AND the Main Feed Regulating Valves begin to throttle closed.
	RO	Places Main Feed Regulating Valve controllers to manual as necessary and controls feed flow when level in SG > 60% or when controller windup is to be removed.
		Places Main Boiler Feed Pump Master Controller back to automatic using SOP 21.1

# SIMULATOR OPERATOR:

Activate trigger 2 when directed by the Lead Examiner to proceed to event 2: VCT Level Transmitter fails low

Page 1 of 1 Op-Test No.: 1 Scenario No.: 4 Event No.: 3 Event Description: VCT Level Transmitter fails low

Time	Position	Applicant's Actions or Behavior
	RO	Diagnoses VCT Level Transmitter failure
	SRO	Enters 2-AOP-CVCS-1, Chemical and Volume Control System Malfunctions, and directs the operator's actions
	ВОР	Holds LCV-112C Control Switch in OPEN When LCV-112C indicates open, then CLOSES LCV-112B
	RO	Places Makeup Control Switch in STOP  If necessary, reduces turbine load to keep Tave on program  When necessary to raise VCT pressure, initiates manual VCT makeup per SOP-3.2, Reactor Coolant System Boron  Concentration Control
	ВОР	Monitors VCT Pressure. Controls VCT pressure 2-5 psig above pre-malfunction (19-20 psig) pressure as follows:  - Coordinates with RO to raise VCT Pressure by manual makeup to VCT per SOP-3.2, Reactor Coolant System Boron Concentration Control  - Lowers VCT Pressure by manually diverting letdown via LCV-112A

### SIMULATOR OPERATOR:

When directed by the Lead Examiner, actuate trigger 3 to cause Faulted Steam Generator, Reactor auto and manual trips fail to actuate

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Op-Test No.: 1 Scenario No. 4: Event No.: 4 Page 1 of 2 Event Description: Faulted Steam Generator, Reactor auto and manual trips fail to actuate			
Time	Position	Applicant's Actions or Behavior	
	RO	Diagnose steam leak	
		Manually actuate Reactor Trip	
	. 1141	Diagnose failure of reactor to trip	
	SRO	Direct team to perform immediate actions of EOP FR-S.1, Response to Nuclear Power Generation/ATWS	
	Critical Task FR-S.1 C	Insert negative reactivity into the core by at least one of the following methods before completing FR-S.1 step 4:  - De-energize the control rod drive MG sets: - Place rod control in manual and insert RCCAs: - Establish emergency boration flow to the RCS	
	SRO	Dispatch NPO to locally trip the reactor	
	RO	Place Rod Control Bank Selector switch in MANUAL Hold Rod Control In-Out Switch in the IN position and manually insert control rods	
	BOP	Manually trip the turbine  Verify AFW pumps running  Start charging pumps  Open MOV-333; Emergency Boration valve  Place both Boric Acid Pumps in high speed  Place running Charging Pump speed controllers in Manual  Open LCV-112B, Suction from RWST  Close LCV-112C, Suction from VCT  Place RCS Makeup Control Switch to STOP  Establish minimum charging flow of 75gpm  Check PRZR Pressure < 2335	
		SIMULATOR OPERATOR: Trip both Rod Drive MG set Output Breakers to insert control rods at this time.	

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Event No.: 4

Scenario No. 4:

Op-Test No.: 1

	Description: Fa	aulted Steam Generator, Reactor auto and manual trips fail to	
Time	Position	Applicant's Actions or Behavior	
	ВОР	Verify Containment Ventilation Isolation	
		Verify Containment Pressure Relief Valves Closed	
	RO	Check reactor trip and turbine trip have occurred	
	ВОР	Maintain total feed flow > 800 gpm until NR level in at least one SG > 10%	
	RO	Verify all dilution paths isolated:	
		<ul> <li>Check FCV-111A demin water flow control valve CLOSED</li> </ul>	
		- Check no flow indicated on FI-111 Primary Water Flow	
		Check for Reactivity Insertion from Uncontrolled RCS Cooldown	
	Critical Task E-2 – A	Isolate the Faulted SG before Transition out of FR-S.1	
	BOP	Isolated Faulted SG - Close all MSIVs	
		<ul> <li>Isolate AFW flow to 22 SG</li> <li>Dispatch NPO to close MS-41, steam supply from 22 SG to 22 ABFP turbine</li> </ul>	
	RO	Isolate Faulted SG  - Identify 22 SG Faulted  - Isolate Main Feed to 22 SG  - Verify 22 SG Atmospheric Steam Dump CLOSED	
	RO	Check CETs < 1200°F  Verify Reactor Subcritical	
	0.00		
	SRO	Return to EOP E-0, Reactor Trip or Safety Injection	

Op-Test No.: 1 Scenario No.: 4 Page 1 of 2 Event No.: 5 Event Description: PORV Fails Open. Block valve used to isolate it.

Time	Position	Applicant's Actions or Behavior	
	RO	Verify Reactor Trip	
	:	Verify Turbine Trip	
	ВОР	Verify power to 480V Buses	
	RO	Verify SI Status	
	ВОР	Performs Attachment 1 while SRO and RO continue in E-0:	
		Verify charging system operation	
		- Starts one charging pump in manual at maximum speed	
		<ul> <li>Align charging system to the RWST (opens LCV-112B, Closes LC-112C, place Makeup Control Switch to STOP)</li> </ul>	
		Check 345 KV MO Disc Switch F7-9 Open	
, mark		- verifes BKR 7 and 9 open	
		Check status of 480 V buses	
		- Dispatches NPO to reset all lighting and MCCs 24A, 27A, and 29A	
	-	- Stops all Condensate Pumps	
		Verify FW Isolation	
		Check if MSIVs should be isolated	
		Check SW system operation	
		Check SI System Operation	
		Check SI system valve aligment	
		Verify Containment Fan Coolers in service	
		Verify AFW Flow to all SGs	
		Verify Containment Ventilation Isolation	
		Verify Containment Isolation Phase A	
		Check if CS status	
		Verify CCR Air Conditioner status	
		Notify SRO Attachment 1 complete	

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Page 2 of 2

Event No.: 5

Scenario No.: 4

Op-Test No.: 1

Event Description: PORV Fails Open. Block valve used to isolate it. Time Position Applicant's Actions or Behavior RO Verifies AFW Pumps running and supplying > 400 GPM AFW flow to the SGs Verifies SI Pump flow and RHR pump flow Checks RCP seal cooling Starts 22 SWP Controls RCS Cool-Down by verifying not dumping steam. reducing AFW flow to >400gpm and closing MSIVs Close the block MOV upstream of the stuck-open PZR PORV Critical Task before completion of the first step in the ERG network that directs E-0 -- M the crew to close the block MOV Check PORVs and Spray Valves: RO PORVs - Closed Determines PORV PCV-455C is open Checks pressure < 2335 Places Control Switch for PCV-455C to CLOSE Determines fPCV-455C is not closed Closes Block Valve 535

#### **EXAMINER NOTE:**

RO

RO

SRO

**TEAM** 

SRO

TEAM

CRS

Terminate the scenario when transition to ES-1.1 is made, or at the discretion of the Lead Examiner

Transition to ES-1.1, SI Termination

Check if RCPs should be stopped

Transitions to E-2, Faulted SG Isolation

Verify that SI Termination Criteria are met

Verify all actions to isolate 22 SG have been completed

Transition to E-1, Loss of Reactor or Secondary Coolant

Checks for any faulted SG

# **Shift Turnover**

# **Watch Team Turnover Sheet:**

Date/Time:

**TODAY** 

Condition:

Power Ops

**RCS Temp:** 

562 °F

% Power:

100%

**RCS Press:** 

2235 psig

**MW Gross:** 

1017

**PZR Level:** 

48 %

**River Water:** 

78 °F

RCS Total Leakage:

0.1 gpm

**Boron Conc:** 

972 ppm

**RCS** Unidentified

0.1 gpm

**Control Rods** 

214 CBD

Leakage:

Equilibrium

Condenser Air leakage

6 SCFM

EFPD:

Xenon:

340

**RCS** Gas activity

2.54e-2 μCi/cc

**PZR Press Control:** 

Channel 1

**PZR Level Control:** 

Channel 2

**Service Water:** 

3 Header Ops

**Risk Assessment:** 

Yellow

Daily Risk Factor:

3.45

### **Plant Equipment Status:**

- 1. EDG 21 is out of service for bearing replacement. It was removed from service 42 hours ago and is due back in 12 hours. Maintenance is currently performing repairs. TS 3.8.1.B surveillance requirement last performed 3 hours ago.
- 2. 21 and 22 EDGs are protected equipment.
- 3. 22 Charging Pump was removed from service for corrective maintenance 18 hours ago. Expected return to service in 35 hours.

#### **Instructions to the Shift:**

Maintain current plant conditions.