November 10, 2004

Mr. Karl W. Singer
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37401-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT UNITS 1 AND 2 — REQUEST FOR ADDITIONAL INFORMATION REGARDING RESPONSE TO NRC BULLETIN 2004-01 (TAC NOS. MC3512 AND MC3513)

Dear Mr. Singer:

By a letter dated July 27, 2004, Tennessee Valley Authority (TVA) provided a response to U.S. Nuclear Regulatory Commission (NRC) Bulletin 2004-01, "Inspection of Alloy 82/182/600 Materials Used in the Fabrication Pressurizer Penetrations and Steam Space Piping Connections at Pressurized-Water Reactors." The NRC staff has reviewed TVA's submittal and finds that additional information is needed before we can complete our review of TVA's response.

This request for additional information was discussed with your staff and a response is requested within 30 days of the receipt of this letter. If you have any questions, please feel free to contact me at (301) 415-1245.

Sincerely,

/RA/

Robert J. Pascarelli, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosure: Request for Additional Information

cc w/encl: See next page

November 20, 2004

Mr. Karl W. Singer
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37401-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT UNITS 1 AND 2 — REQUEST FOR ADDITIONAL INFORMATION REGARDING RESPONSE TO NRC BULLETIN 2004-01 (TAC NOS. MC3512 AND MC3513)

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REQUEST FOR ADDITIONAL INFORMATION

NRC BULLETIN 2004-01

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-327 AND 50-328

The U.S. Nuclear Regulatory Commission (NRC) staff has completed its review of your response, dated July 27, 2004, to NRC Bulletin 2004-01, "Response to NRC Bulletin 2004-01: Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at Pressurized-water Reactors." Based on the staff's review, please provide a supplemental response which addresses the following question.

Your response to Bulletin 2004-01 Question (1)(c) did not clearly communicate your intentions with respect to ensuring that an appropriate dialogue would be established with NRC technical staff in the event that circumferential primary water stress corrosion cracking (PWSCC) is identified at any locations covered under the scope of Bulletin 2004-01. The NRC staff addressed this issue, in part, on page 5 of Bulletin 2004-01 stating, ". . . the NRC staff believes that the topic of NDE [nondestructive examination] scope expansion should be discussed with the NRC if circumferential PWSCC is observed in either the pressure boundary or non-pressure boundary portions of any locations covered under the scope of this bulletin to ensure that the licensee has performed an adequate extent-of-condition evaluation."

Because of the potential plant-specific and generic significance of circumferential PWSCC at locations covered under the scope of Bulletin 2004-01, it is the NRC staff's position that cognizant members of the Office of Nuclear Reactor Regulation's Materials and Chemical Engineering Branch (EMCB) should be promptly made aware of any emerging issue regarding this degradation phenomenon at your facility. This is important not only for the reason cited in the passage above, from Bulletin 2004-01, but also so that the NRC staff can evaluate any such information and fulfill its obligation to inform other U.S. nuclear power plant licensees of new operational experience which may be relevant to the continued safe operation of their facilities.

It is the NRC staff's expectation that, if you obtain inspection results in the future that indicate that circumferential PWSCC may be occurring at any location covered under the scope of Bulletin 2004-01, you should contact your NRC Headquarters Project Manager (PM) and request a teleconference or meeting with EMCB technical staff. Notification of your NRC PM should allow ample time for you to incorporate any insights from the aforementioned teleconference or meeting into your plans for evaluating the extent of condition at your facility prior to the end of the outage during which the degradation was discovered.

ENCLOSURE

In order to document TVA's intent to follow the guidance stated above, please provide a supplement to your Bulletin 2004-01 item (1)(c) response which states:

> If circumferential cracking is observed in either the pressure boundary or non-pressure boundary portions of any locations covered under the scope of this bulletin, [we] will develop plans to perform an adequate extent-of-condition evaluation and [we] will discuss those plans with cognizant NRC technical staff prior to restarting the affected unit.

Mr. Karl W. Singer
Tennessee Valley Authority

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SEQUOYAH NUCLEAR PLANT

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