November 1, 2004

Mr. James F. Klapproth, Manager
Engineering & Technology
GE Nuclear Energy
175 Curtner Avenue
San Jose, CA 95125

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION – LICENSING TOPICAL REPORT NEDC-32983P-A, "GENERAL ELECTRIC METHODOLOGY FOR REACTOR PRESSURE VESSEL FAST NEUTRON FLUX EVALUATION" (TAC NO. MC3788)

Dear Mr. Klapproth:

By letters dated July 14 and September 10, 2004, GE Nuclear Energy (GENE) provided information to support the removal of limitations regarding BWR shroud fluence calculations in the subject licensing topical report. The NRC staff has reviewed this information and has prepared the enclosed request for additional information (RAI). This request was discussed with Mike Lalor of your staff and it was agreed that a response would be provided within 60 days of receipt of this letter.

If you have any questions regarding this RAI, please contact me at (301) 415-1445.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Project No. 710

Enclosure: Request for Additional Information

cc w/encl: See next page

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Enclosures: Request for Additional Information

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REQUEST FOR ADDITIONAL INFORMATION

LICENSING TOPICAL REPORT NEDC-32983P-A, "GENERAL ELECTRIC METHODOLOGY FOR REACTOR PRESSURE VESSEL FAST NEUTRON FLUX EVALUATION"

GE NUCLEAR ENERGY (GENE)

PROJECT NO. 710

1. By letter dated July 14, 2004, GENE provided information and requested removal of the limitations regarding vessel fluence calculations in licensing topical report (TR) NEDC-32983P-A. The U.S. Nuclear Regulatory Commission (NRC) staff requests that GE provide the measured and calculated values in Figure A of the July 14, 2004, letter in a tabular form.

2. By letter dated September 10, 2004, GENE, supplemented the July 14, 2004, letter and provided additional information to support its request for removal of limitations regarding boiling water reactor (BWR) shroud fluence calculations in the licensing TR. Due to the limited information provided in the submittal, the NRC staff requests the following additional information:

   a. Please clarify the purpose of this submittal and the basis for any request.

   b. The main issue with the existing shroud calculations in the NEDC-32983P-A topical is the axial bias of the C/M values. The submittal indicates a radial dependance of the samples but does not indicate that there is elevation dependence of the samples. Given that the axial shroud weld cracking, attributed to irradiation assisted stress corrosion, is an important BWR issue, how do you justify lifting the limitation?

   c. The samples were taken from two BWR4 plants H and V. Why is it appropriate to generalize these results to all BWRs?

   d. The currently submitted data were from samples irradiated in the early 1990s, why was the data not part of the initial submittal? Are the old data to be ignored?