

298

CONDITION REPORT
ED 8342-1

BOE 77 0013 76

NO. 2000-0781

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EVENT DATE	EVENT TIME	DISCOVERY DATE	DISCOVERY TIME	REFERENCE DOCUMENTS/ASSET NUMBER
April 5 2000	20:00	April 5, 2000	20:00	WO 09-00320-00/T-1

CONDITION DESCRIPTION

This Condition Report identifies a potential non-compliance issue against the ASME code. During a VT-2 pressure test examination of the reactor vessel head, leakage from the CRD structure blocked the visual examination of the reactor vessel head studs. To examine the reactor vessel head bolting, the boric acid residue would need to be removed, however this would invalidate the requirements of the current relief request. The ASME Code would require insulation removal during Mode 3, approved Relief Request A7 allows examination for evidence of leakage during Mode 5 or 6. This was the reason for the examination.

The 1986, ASME Code, Paragraph IWA-5241, requires the owner to examine the exposed surface for evidence of leakage. Paragraph IWA 5250(2) states "if leakage occurs at a bolted connection the bolting shall be removed, VT-3 visually examined for corrosion, and evaluated." Because leakage from the CRDs caused boric acid residue to cover the inner side of all the reactor head bolts then a VT-2 examination could not be completed. Cleaning of the head vessel studs will remove any evidence of leakage. By default, a VT-3 bolting examination is required.

This Condition Report will track and resolve this ASME non-conformance issue. MDT 2254 is written to perform a VT-3 exam on all the reactor vessel head bolting. Until the condition is determined to be irrelevant, (e.g., the bolting is determined to be satisfactory via the VT-3 exam) then this is a Mode 5 to Mode 4 restraint.

CONTINUED

NAME (Print)	SIGNATURE	DATE	ORGANIZATION	TELEPHONE NO	MAIL LOCATION
Peter J. Senjuk	<i>Peter J. Senjuk</i>	April 6, 2000	PETP	8377	1056

SUPERVISOR

PLANT OPERATIONS REVIEW YES NO

COMMENTS

Pulling Rx Head Studs for Refueling, no increase in SLIP.

RECOMMENDED CATEGORY Routine

CONTINUED

NAME (Print)	SIGNATURE	DATE	ORGANIZATION	TELEPHONE NO	MAIL LOCATION
Robert BISHEL	<i>Robert Bishel</i>	4/6/00	PETP	7918	1056

PLANT OPERATIONS

REPORTABILITY: 1 HR 4 HR 24 HR N/A

OPERABILITY: YES NO N/A

IMMEDIATE ACTIONS TAKEN OR NEEDED / COMMENTS

EVALUATE FOR MODE 5 RESTRAINT

PLANT ENGINEERING TO BE NOTIFIED PER BORIC ACID CORROSION CONTROL PROGRAM

cc: Initiator
CR Files
Nuclear Records Management

CONTINUED

NAME (Print)	SIGNATURE	DATE	TIME
VINEGAR	<i>V. Phogyn</i>	4/6/00	0320

110673 / 0001 / 6-21

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OWNER PETP		CATEGORY ROUTINE		DUE DATE 6/5/00		OPERATING EXPERIENCE REPORT: <input type="checkbox"/> EVALUATE <input type="checkbox"/> INITIATE	
<input type="checkbox"/> CATPR	CAUSE DETERMINATION <input checked="" type="checkbox"/> APPARENT <input type="checkbox"/> ROOT CAUSE <input type="checkbox"/> MULTI-DISC. ROOT CAUSE			<input type="checkbox"/> ERB			
<input type="checkbox"/> EXPERIENCE REVIEW	<input type="checkbox"/> EXTENT OF CONDITION	<input type="checkbox"/> POTENTIAL MRF	<input type="checkbox"/> OTHER REVIEW REQUIRED				

MRC COMMENTS

MODE 4 RESTRAINT

Rm. W

SUPERVISOR ASSIGNED DUE DATE:		<input type="checkbox"/> CONTINUED	
10 CFR PART 217 <input checked="" type="checkbox"/> NO <input checked="" type="checkbox"/> YES	IF YES, DATE	SYSTEM CAPABLE OF PERFORMING SPECIFIED FUNCTION? <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO <input type="checkbox"/> N/A	
ANI REVIEW REQUIRED? <input type="checkbox"/> NO <input checked="" type="checkbox"/> YES IF YES, ANI SIGNATURE REQUIRED	SIGNATURE <i>[Signature]</i>	DATE 4-26-00	
PREPARER (Print) PETER J. SENELM	SIGNATURE <i>Peter J. Senelm</i>	DATE 4/26/00	
SUPERVISOR APPROVAL (Print) ALLEN L McALLISTER	SIGNATURE <i>[Signature]</i>	DATE 4/27/00	
MANAGER APPROVAL (Print) <input checked="" type="checkbox"/> N/A	SIGNATURE	DATE	
DIRECTOR APPROVAL (Print) <input checked="" type="checkbox"/> N/A	SIGNATURE	DATE	
VICE PRESIDENT APPROVAL (Print) <input checked="" type="checkbox"/> N/A	SIGNATURE	DATE	
ERB APPROVAL (Print) <input type="checkbox"/> OE REQUIRED <input checked="" type="checkbox"/> N/A	SIGNATURE	DATE	
ERB APPROVAL (Print) <input checked="" type="checkbox"/> N/A	SIGNATURE	DATE	

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CONTINUATION

The reactor vessel flange bolting is scheduled for VT-3 bolting examination. WO 00-1588-00 will examine all sixty studs. These studs are removed and located in the R/A train bay. Quality Control will perform the VT-3 bolting examination. This work is scheduled to be performed between April 19, 2000 and April 28, 2000

The reason for VT-3 examination was that dry BA from the CRD structure blocked the upper inside bolting area for examination. One could contend that the flange area inspection was all that was required. Because the VT-2 examiner determined that an adequate examination could not be performed and that the vessel bolting was to be removed, then the performance of the VT-3 would not be significant work issue.

Only evidence of leakage from the CRD structure was identified. This evidence from the CRD structure does not effect operability to the reactor vessel flange. The system was pressurized, no wetted areas where found. Based upon amount of leakage during Cycle 12, the system will be capable of performing its safety function. Overall, system leakage from the RCS system was within Technical Specification requirements. CR 2000-782 will address CRD flange leakage.

After the above work is documented, e.g., Quality Control signs the VT-3 examination sheet then the Mode 5 to 4 restraint can be removed. This CR will require ANI review.

Peter J. Seniuk / 4/17/00
 Preparer: Peter J. Seniuk / Date

Allen McAllister / 4/17/00
 Supervisor: Allen McAllister / Date

006-57-001A 270

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CONTINUED

The reactor vessel flange bolting VT-3 examination is complete. WD 00-1588-00 was performed on 4/19/00, all 60 studs had no evidence of corrosion. The above work is documented, e.g., Quality Control has signs the VT-3 examination sheet. Therefore, this item is not a Mode 5 to 4 restraint.

This CR requires ANI review because it effects an ASME component.

ANI Review: *PJ* Date: 4-24-00

Peter J. Seniuk , 4/22/00
Preparer: Peter J. Seniuk / Date

Allen McAllister , 4/24/00
Supervisor: Allen McAllister / Date

*Note: Cause of this C.R. was O.P. leakage from Control Rod Drive flanges.
See response to CR 2000-0782 for cause and leakage
correction. *PJ* 4/26/00*

CR 2000-0781
5055

5-BESSH NUCLEAR POWER STATION

VISUAL EXAMINATION DATA SHEET

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FIGURE NO. <u>NA</u>		QC FILE NO. <u>DD-001587-000-ME-01</u>		
COMPONENT ID <u>Rx VESSEL STUDS (Ln)</u>		<input checked="" type="checkbox"/> DIRECT <input type="checkbox"/> REMOTE		
SYSTEM <u>Rx COOLANT</u>		EQUIPMENT USED <u>FLASHLIGHT</u>		
DRAWING NO. <u>ISI-SK-00412</u>		<input type="checkbox"/> VIDEOTAPE <input type="checkbox"/> PHOTOGRAPH		
PROCEDURE NA-QC-05560 REV <u>02 C-1</u>		TAPE/REEL NO. <u>NA</u>		
TYPE EXAM: <input type="checkbox"/> VT-1 <input checked="" type="checkbox"/> VT-3		LIGHT METER NO. <u>NA</u> CDD: <u>NA</u>		
INDICATIONS	SAT	UNSAT	N/A	DESCRIPTION/GENERAL COMMENTS
1. Structural Deformation or Degradation			✓	
2. Missing/Detached/Loose Items			✓	
3. Cracked or Fractured Items			✓	
4. Corrosion	✓			NO EVIDENCE OF CORROSION
5. Erosion/Water			✓	
6. Damaged Threads			✓	
7. Structural Distortion/Displacement			✓	
8. Crack-Like Flaws			✓	
9. Foreign Material Accumulation			✓	
10. Others			✓	
EXAMINER <u>[Signature]</u>		LEVEL DATE <u>4/15/00</u>		
LEVEL III <u>[Signature]</u>		LEVEL DATE <u>4/14/00</u>		
PLANT ENGR <u>[Signature]</u>		LEVEL DATE <u>4-21-00</u>		
EXAMINER <u>C. Steven Stigall III</u>		LEVEL DATE <u>4/19/00</u>		
ANI _____		PQA/R/WORK REQUEST NO. <u>NA</u>		

[1774]

RECORD END SHEET
No. 140

000737.001772

**END
OF
RECORD**