

October 15, 2004

LICENSEE: Indiana Michigan Power Company
FACILITY: Donald C. Cook Nuclear Plant, Units 1 and 2
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON
SEPTEMBER 16, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY
COMMISSION AND INDIANA MICHIGAN POWER COMPANY, PERTAINING
TO THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE
RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Indiana Michigan Power Company (I&M) held a telephone conference call on September 16, 2004, to discuss and clarify a request for additional information (RAI) concerning the Donald C. Cook Nuclear Plant, Units 1 and 2, license renewal application (LRA). The conference call was useful in clarifying the intent of the staff's RAI.

Enclosure 1 provides a listing of the telephone conference call participants. Enclosure 2 contains the RAI discussed with the applicant, including a brief description on the status of the RAI.

The applicant has had an opportunity to comment on this summary.

/RA/

Jonathan G. Rowley, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

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| OFFICE | PM:RLEP | LA:RLEP | SC:RLEP |
| NAME | JRowley | YEdmonds | SLee |
| DATE | 10/15/04 | 10/15/04 | 10/15/04 |

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RLEP RF
J. Rowley (PM)

E-MAIL:

RidsNrrDrip
RidsNrrDe
G. Bagchi
K. Manoly
W. Bateman
J. Calvo
R. Jenkins
P. Shemanski
J. Fair
RidsNrrDssa
RidsNrrDipm
D. Thatcher
R. Pettis
C. Li
M. Itzkowitz (RidsOgcMailCenter)
R. Weisman
M. Mayfield
A. Murphy
S. Smith (srs3)
S. Duraiswamy
Y. L. (Renee) Li
RLEP Staff

R. Gramm
A. Howell
J. Stang
J. Strasma, RIII
M. Kotzalas
OPA
NRR/ADPT secretary (RidsNrrAdpt)

LIST OF PARTICIPANTS FOR TELEPHONE CONFERENCE CALL
TO DISCUSS THE DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
SEPTEMBER 16, 2004

Participants

Jonathan Rowley
Hansraj Ashar
Ralph Schlichter

Affiliations

U.S. Nuclear Regulatory Commission (NRC)
NRC
Indiana Michigan Power Company

**REQUEST FOR ADDITIONAL INFORMATION
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION**

SEPTEMBER 16, 2004

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Indiana Michigan Power Company (I&M) held a telephone conference call on September 16, 2004, to discuss and clarify the Donald C. Cook Nuclear Plant, Units 1 and 2 (Cook), license renewal application (LRA). The following RAI was discussed during the telephone conference call.

RAI 3.5-1

In line item 3.5.1-3 of Table 3.5.1 of the LRA, the applicant indicates that the aging effects related to loss of material due to corrosion of bellows, and dissimilar metal welds are managed consistent with NUREG-1801. NUREG-1801 recommends the examination of penetration bellows and the associated dissimilar welds based on the operating experience with the stress corrosion cracking of bellows as documented in NRC Information Notice 92-20. The applicant is requested to provide the following information related to the examination and/or testing of containment penetration bellows:

How many penetration bellows are in the Cook containments? Please summarize the operating experience related to the examination of these bellows. If provisions are made to assess their leaktightness (as they are not accessible for visual examination), please provide a summary of these provisions (including frequency of tests), and indicate if such leaktightness assessment of the bellows is part of the LRA aging management program (AMP) B.1.15 or B.1.8.

Additional Information Requested

I&M's supplemental response to RAI 3.5-1 indicates that the Cook penetration bellows do not provide a containment pressure-boundary function, they are not included in the scope of license renewal, and are not subject to aging management. The staff had concerns that cracking in two sets of bellows in the penetration could jeopardize containment isolation and that the bellows do perform support functions for two safety-related pipes and structures. The applicant is requested to justify why these bellows should not be in the scope of license renewal.

Discussion

The applicant directed the staff to Figure 5.2-4 of the Cook updated final safety analysis report (UFSAR). Figure 5.2-4 illustrates the fuel transfer tube on which the bellows in question are located. I&M used the figure to confirm how the bellow inside containment is completely independent of those bellows outside containment and that it was impossible for there to be communication between the inside and outside bellows. I&M used the figure to confirm that the bellows do not perform support functions. The staff will evaluate this information and no further I&M action is requested at this time.

Enclosure 2

Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
2443 Warrenville Road, Suite 210
Lisle, IL 60532-4351

Township Supervisor
Lake Township Hall
P.O. Box 818
Bridgman, MI 49106

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
7700 Red Arrow Highway
Stevensville, MI 49127

David W. Jenkins, Esquire
Indiana Michigan Power Company
One Cook Place
Bridgman, MI 49106

Mayor, City of Bridgman
P.O. Box 366
Bridgman, MI 49106

Special Assistant to the Governor
Room 1 - State Capitol
Lansing, MI 48909

Mr. John A. Zwolinski
Director, Design Engineering and
Regulatory Affairs
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

Laura Kozak
2443 Warrenville Rd.
Lisle, IL 60532

Michigan Department of Environmental
Quality
Waste and Hazardous Materials Div.
Hazardous Waste & Radiological
Protection Section
Nuclear Facilities Unit
Constitution Hall, Lower-Level North
525 West Allegan Street
P.O. Box 30241
Lansing, MI 48909-7741

Michael J. Finissi, Plant Manager
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

Mr. Joseph N. Jensen, Site Vice President
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

Mr. Fred Emerson
Nuclear Energy Institute
1776 I Street, N.W., Suite 400
Washington, DC 20006-3708

Richard J. Grumbir
Project Manager, License Renewal
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

Mr. Mano K. Nazar
Senior Vice President and Chief Nuclear
Officer
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107