

October 8, 2004

LICENSEE: Tennessee Valley Authority

FACILITY: Watts Bar Nuclear Power Plant, Unit 1.

SUBJECT: SUMMARY OF AUGUST 26, 2004, MEETING WITH
TENNESSEE VALLEY AUTHORITY AND WESTINGHOUSE, ON STEAM
GENERATOR REPLACEMENT AT WATTS BAR NUCLEAR PLANT, UNIT 1
(TAC NO. MC3937)

On August 26, 2004, a Category 1 public meeting was held between the U.S. Nuclear Regulatory Commission (NRC) and representatives of Tennessee Valley Authority (TVA, licensee), Westinghouse, and Bechtel at NRC Headquarters, Two White Flint North, 11555 Rockville Pike, Rockville, Maryland. The purpose of the meeting was to discuss the licensee's upcoming plans to replace existing steam generators (SGs) at Watts Bar Nuclear Plant (WBN), Unit 1, with new SGs.

A list of attendees is provided in Attachment 1 and the licensee's handouts provided at the meeting are included in Attachment 2.

Mr. Paul Trudel, the licensee's representative, gave a brief presentation indicating that Sequoyah Nuclear Plant (SQN) Unit 1 recently replaced SGs and these lessons learned will be incorporated at WBN Unit 1. The licensee also explained similarities and differences from the SQN Unit 1 SG replacement process. Mr. Trudel presented salient features of the replacement SGs and compared it to the existing units. He discussed major activities and the status of the current projects associated with SG replacements.

The licensee discussed changes to the specific design and operational areas requiring NRC approval. The most notable changes include the bar lock mechanical couplers, opening of penetrations in the shield building dome, and nuclear steam supply system coolant reanalysis. The licensee provided a list of computer codes that they intend to use for reactor coolant loop reanalysis which were the same computer codes that were approved by the NRC staff for the AP600 piping system analysis. The licensee provided a potential schedule of license amendment submittals and proposed 10 CFR 50.59 changes and the evaluations to support the replacement of the SGs.

CONTACT: Robert Pascarelli, NRR
(301) 415-1245

During the meeting, the NRC staff provided high-level comments concerning quality assurance related to SG tubes and revisions to the Updated Final Safety Analyses Report.

Mr. Trudel indicated that Westinghouse has a full-time staff at the site where the SG tubes are being fabricated to check quality and assure proper shipment. The licensee intends to update and reanalyze the complete chapter on design-basis accidents.

No members of the public attended this meeting. There were no public comments received.

Please direct any inquiries to Robert Pascarelli, Project Manager at (301) 425-1245, or MMC1@NRC.gov

/RA/

Robert J. Pascarelli, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-390

Attachments: 1. List of Attendees
2. Licensee Handout

cc w/atts: See next page

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cc w/atts: See next page

Distribution: See next page

Package:

Meeting Notice Accession No: ML042290128

Meeting Summary Accession No.: ML042920039

Handouts Accession No.: Attachment 1: ML042920182 Attachment 2: ML042920186

ADAMS Accession No.: ML042920039 NRC-001

OFFICE	PDII-2/PM	PDII-2/SC	
NAME	RPascarelli	MMarshall	
DATE	10/8/04	10/8/04	

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