PSEG Nuclear LLC
P.O. Box 236, Hancocks Bridge, New Jersey 08038-0236

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PSEG Nuclear LLC

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

LCR S03-05, IMPLEMENTATION OF ALTERNATE SOURCE TERM
SALEM GENERATING STATION, UNITS 1 AND 2
FACILITY OPERATING LICENSE DPR-70 AND DPR-75
DOCKET NOS. 50-272 AND 50-311

Re: TRANSMITTAL OF PSEG CALCULATIONS

On April 26, 2004, PSEG Nuclear, LLC (PSEG) submitted a request for revision of Appendix A of the Technical Specifications for the Salem Nuclear Generating Station, Units 1 and 2 implementing the guidelines provided by Regulatory Guide 1.183, "Accident Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Stations".

In the submittal, the NRC was informed that PSEG had completed calculations to support the revised dose analysis and that the calculations are available for the staff's review upon request. During the working meeting held on September 9, 2004 at the NRC headquarters, NRC staff requested copies of the applicable calculations in order to complete their review of our submittal.

The following calculations are included for NRC staff review:

1. S-C-ZZ-MDC-1945, Post LOCA EAB, LPZ and CR Doses – Alternative Source Term (AST), Rev. OIR1
2. S-C-ZZ-MDC-1948, EAB, LPZ and CR Doses, Rod Ejection Accident (REA)-AST, Rev. OIR2
3. S-C-ZZ-MDC-1949, EAB, LPZ and CR Doses, Steam Generator Tube Rupture (SGTR) Accident-AST, Rev. OIR2
4. S-C-ZZ-MDC-1950, EAB, LPZ and CR Doses, Main Steam Line Break (MSLB) Accident-AST, Rev. OIR1
5. S-C-ZZ-MDC-1951, EAB, LPZ and CR Doses, RCP Locked Rotor Accident (LRA)-AST, Rev. OIR1
6. S-C-ZZ-MDC-1959, CR χ/Qs Using ARCON 96 Code-Non-LOCA Releases, Rev. OIR1
7. S-C-ZZ-MDC-1987, Input Parameters for Salem AST Dose Calcs, Rev. 1 (Containment Spray Coverage Evaluation)

Diskettes Not Included A001

8. S-C-ZZ-MEE-1805, Waste Gas Decay Tank and Volume Control Tank Rupture Accidents-AST Analysis, Rev. 0 (w/o attachments)
   Note: This evaluation was completed to facilitate the adoption of AST TEDE terminology only and does not represent any changes to the Salem Units 1 and 2 current design/licensing basis.

Additionally, at NRC staff's request, a CD containing the design inputs and RADTRAD data used to develop our submittal is included with this package.

This submittal does not contain any information considered "Proprietary" or that should be withheld from public disclosure.

If you have any questions or require additional information, please contact Mr. Jesus Arias at (856) 339-5245.

Sincerely,

Christina Perino
Director- Licensing and Nuclear Safety

C: Mr. S. Collins, Administrator – Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. D. Collins, Project Manager - Salem & Hope Creek
U. S. Nuclear Regulatory Commission
Mail Stop 08C2
Washington, DC 20555

USNRC Senior Resident Inspector – Salem Units 1 and 2

Mr. K. Tosch, Manager IV
Bureau of Nuclear Engineering
PO Box 415
Trenton, New Jersey 08625