

September 29, 2004

Mr. Mano K. Nazar
Senior Vice President and Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

SUBJECT: REQUESTS FOR ADDITIONAL INFORMATION FOR THE REVIEW OF
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL
APPLICATION

Dear Mr. Nazar:

By letter dated October 31, 2003, Indiana Michigan Power Company submitted an application pursuant to Title 10 of the *Code of Federal Regulations* Part 54 (10 CFR Part 54), to renew the operating licenses for the Donald C. Cook Nuclear Plant (CNP), Units 1 and 2, for review by the U.S. Nuclear Regulatory Commission (NRC). The NRC staff is reviewing the information contained in the license renewal application (LRA) and has identified, in the enclosure, areas where additional information is needed to complete the review.

Based on discussions with Mr. Richard Grumbir of your staff, a mutually agreeable date for your response is within 30 days from the date of this letter. If you have any questions regarding this letter or if circumstances result in your need to revise the response date, please contact me at 301-415-4053 or by e-mail at jgr@nrc.gov.

Sincerely,

/RA/

Jonathan Rowley, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosure: As stated

cc w/encl: See next page

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Donald C. Cook Nuclear Plant, Units 1 and 2

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**DONALD C. COOK NUCLEAR PLANT (CNP) , UNITS 1 AND 2
LICENSE RENEWAL APPLICATION (LRA)
REQUESTS FOR ADDITIONAL INFORMATION (RAIs)**

RAI 3.1.2-4

In CNP LRA Section 3.1.2.2.6 and Table 3.1.1-11, the applicant credits CNP AMP B.1.27, "Reactor Vessel Internals Plates, Forgings, Welds, and Bolting Program," to manage changes in dimension due to void swelling of reactor vessel internals.

In CNP LRA Table 3.1.2-2, Page 3.1-51, the applicant credits the Reactor Vessel Internals (CASS) Program to manage distortion of the lower support plate and lower core plate support column cap in a treated water environment. Additionally, in LRA Appendix B, Sections B.1.27, "Reactor Vessel Internals Plates, Forgings, Welds, and Bolting Program," and B.1.28, "Reactor Vessel Internals Cast Austenitic Stainless Steel," the applicant describes the program description for each as managing the aging effect of distortion due to void swelling.

The staff requests the applicant to update LRA Section 3.1.2.2.6 and Table 3.1.1 item number 11 to include the Reactor Vessel Internals (CASS) Program with managing this aging effect or justify why it should not be credited with managing this aging effect.

Enclosure