Definitions 1.1

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1.1 Definitions (continued)

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CHANNEL CHECK	A CHANNEL CHECK shall be the qualitative assessment, by observation, of channel behavior during operation. This determination shall include, where possible, comparison of the channel indication and status to other indications or status derived from independent instrument channels measuring the same parameter.
CHANNEL OPERATIONAL TEST (COT)	A COT shall be the injection of a simulated or actual signal into the channel as close to the sensor as practicable to verify the OPERABILITY of required alarm, interlock, display, and trip functions. The COT shall include adjustments, as necessary, of the required alarm, interlock, and trip setpoints so that the setpoints are within the required range and accuracy.
CORE ALTERATION	CORE ALTERATION shall be the movement of any fuel, sources, or reactivity control components, within the reactor vessel with the vessel head removed and fuel in the vessel. Suspension of CORE ALTERATIONS shall not preclude completion of movement of a component to a safe position.
CORE OPERATING LIMITS REPORT (COLR)	The COLR is the unit specific document that provides cycle specific parameter limits for the current reload cycle. These cycle specific parameter limits shall be determined for each reload cycle in accordance with Specification 5.6.5. Plant operation within these limits is addressed in individual Specifications.
DOSE EQUIVALENT I-131	DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcuries/gram) that alone would produce the same dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The dose conversion factors used for this calculation shall be those listed under the "Effective" column of Table 2.1 of Federal Guidance Report 11.
È-AVERAGE DISINTEGRATION ENERGY	È shall be the average (weighted in proportion to the concentration of each radionuclide in the reactor coolant at the time of sampling) of the sum of the average beta and gamma energies per disintegration (in MeV) for isotopes, other than

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RCS Operational LEAKAGE 3.4.13

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3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.13 RCS Operational LEAKAGE

- LCO 3.4.13 RCS operational LEAKAGE shall be limited to:
 - a. No pressure boundary LEAKAGE;

- b. 1 gpm unidentified LEAKAGE;
- c. 10 gpm identified LEAKAGE;
- 0.3 gpm total primary to secondary LEAKAGE through all steam generators (SGs); and
- e. 150 gallons per day primary to secondary LEAKAGE through | any one SG.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

	CONDITION	REQUIRED ACTION		COMPLETION TIME	
A.	RCS LEAKAGE not within limits for reasons other than pressure boundary LEAKAGE.	A.1	Reduce LEAKAGE to within limits.	4 hours	
Β.	Required Action and associated Completion Time of Condition A not met.	B.1 AND	Be in MODE 3.	6 hours	
OR	B.2	Be in MODE 5.	36 hours		
	Pressure boundary LEAKAGE exists.				

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RCS Specific Activity 3.4.16

- 3.4 REACTOR COOLANT SYSTEM (RCS)
- 3.4.16 RCS Specific Activity
- LCO 3.4.16 The specific activity of the reactor coolant shall be within limits.
- APPLICABILITY: MODES 1 and 2, MODE 3 with RCS average temperature $(T_{avg}) \ge 500^{\circ}F$.

ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME	
Α.	DOSE EQUIVALENT I-131 > 0.25 μ Ci/gm.	Note LCO 3.0.4 is not applicable.			I
		A.1	Verify DOSE EQUIVALENT I-131 \leq 60 μ Ci/gm.	Once per 4 hours	ł
		AND			
		A.2	Restore DOSE EQUIVALENT I-131 to within limit.	48 hours	
В.	Gross specific activity of the reactor coolant not within limit.	B.1	Be in MODE 3 with Tavg < 500°F.	6 hours	

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RCS Specific Activity 3.4.16

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ACTIONS (continued)

	CONDITION	REQUIRED ACTION		COMPLETION TIME
C.	Required Action and associated Completion Time of Condition A not met.	C.1	Be in MODE 3 with Tavg < 500°F.	6 hours
	OR			
	DOSE EQUIVALENT I-131 > 60 μ Ci/gm.			

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SURVEILLANCE REQUIREMENTS

	FREQUENCY	
SR 3.4.16.1	Verify reactor coolant gross specific activity \leq 100/ E μ Ci/gm.	7 days
SR 3.4.16.2	NOTE Only required to be performed in MODE 1. Verify reactor coolant DOSE EQUIVALENT I-131 specific activity $\leq 0.25 \ \mu \text{Ci/gm}$.	14 days AND Between 2 and 6 hours after a THERMAL POWER change of ≥ 15% RTP within a 1 hour period
		(continued)

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HBRSEP Unit No. 2

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RCS Specific Activity 3.4.16

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HBRSEP Unit No. 2

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