

September 23, 2004

Mr. Stephen M. Quennoz, Vice President
Power Supply/Generation
Portland General Electric Company
Trojan Nuclear Plant
71760 Columbia River Highway
Rainier, Oregon 97048

SUBJECT: REVIEW OF FINAL STATUS SURVEY REPORT FOR THE MAIN STEAM
SUPPORT STRUCTURE, ELECTRICAL PENETRATION AREA AND STEAM
GENERATOR BLOWDOWN BUILDING AT THE TROJAN NUCLEAR PLANT

Dear Mr. Quennoz:

On March 30, 2004, Portland General Electric Company (PGE) submitted the Final Status Survey Report (FSSR) for the Trojan Nuclear Plant (TNP) Main Steam Support Structure, Electrical Penetration Area and Steam Generator Blowdown Building. The U.S. Nuclear Regulatory Commission (NRC) staff has completed its review of the FSSR and concludes that: (1) the final status survey was conducted in accordance with the License Termination Plan (LTP); (2) the FSSR contains the information identified in NUREG-1757, "Consolidated NMSS Decommissioning Guidance," Section 4.5; and (3) the final status survey results demonstrate that the Main Steam Support Structure, Electrical Penetration Area and Steam Generator Blowdown Building meet the radiological criteria for unrestricted release identified in the LTP.

However, the staff would like PGE to consider clarifying the FSSR to address the following review comments:

1. Section 2.1.1, page 2-1, paragraph 2, sentences 6 & 7 - The sentences, as currently written, can be misinterpreted to mean that some surfaces were remediated and characterized, and the resultant post-remediation characterization data was used for classifying an area, rather than reflecting the pre-remediation condition. Table 2-1 presented on page 2-12 does provide clarification in a footnote which states that pre-remediation data were used to classify areas. However, the staff recommends that PGE consider rewording the sentences on page 2-1 to reflect that pre-remediation data served as the basis for survey unit classification only.

2. Appendix C, page C-27 - The maximum reported scan measurement value was 20,090 dpm/100 cm² for survey unit S18059A, which is close to the 21,000 dpm/100cm² investigation level. The investigation summary notes that no investigation was performed in the survey unit because of no anomalous data. The surface scan map provided on page C-23 of the Supplement to Appendix C has a notation for an instrument alarm for scan location 085, which corresponds to both the survey unit maximum value on page C-27 and the reported value of 20,090 dpm/100 cm² listed on page C-20 of the Supplement to Appendix C. Please provide clarification whether this location was investigated, including whether a static measurement was performed to quantify the residual radioactivity.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, and your response (if any) will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

If you have any questions concerning this evaluation, please contact me at (301) 415-6607.

Sincerely,

/RA/

John T. Buckley, Project Manager
Decommissioning Directorate
Division of Waste Management
and Environmental Protection
Office of Nuclear Material Safety
and Safeguards

Docket No.: 50-344
License No.: NPF-1

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