



UNITED STATES  
**NUCLEAR REGULATORY COMMISSION**  
REGION I  
475 ALLENDALE ROAD  
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

September 20, 2004

Mr. Gregory A. Maret  
Vice President - Decommissioning  
Yankee Atomic Electric Company  
49 Yankee Road  
Rowe, Massachusetts 01367

SUBJECT: NRC INSPECTION REPORT NO. 50-029/2004-001

Dear Mr. Maret:

On June 24, 2004, the NRC completed an inspection at your nuclear reactor facility in Rowe, Massachusetts, which covered an inspection period that began on January 22, 2004. The findings of the inspection were discussed with you and members of your staff on August 5, 2004. The enclosed report presents the results of that inspection.

Your radiation protection, radioactive waste shipping, quality assurance and self-assessment, and maintenance and operations surveillance programs were inspected during this inspection period. In addition, management changes in your site organization were reviewed as related to the safe completion of decommissioning activities. The inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observations by the inspector. Effective programs for maintaining personnel exposures to radiation as low as is reasonably achievable (ALARA) and conducting proper audits and surveillances were noted. However, there were two self-identified issues associated with shipments of building demolition and construction debris which did not meet NRC requirements.

Based on the results of this inspection, the NRC has determined that two Severity Level IV violations of NRC requirements occurred during this inspection period. The violations are: 1) exceeding the free standing liquid criteria of 10 CFR 61.56 (a) (3) in a radioactive waste shipment of demolition and construction debris, and 2) noncompliance with the transportation requirements of 49 CFR 393.102 (a), as required by 10 CFR 71.5 (a), causing a container of demolition and construction debris to slide off its trailer onto a public roadside. These violations are being treated as Non-Cited Violations (NCVs), consistent with Section VI.A of the NRC Enforcement Policy. The NCVs are described in the subject inspection report. If you contest any of the violations or severity level of the NCVs, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001; with copies to the Regional Administrator, Region I; and the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington, DC 20555-0001.

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2

In accordance with Section 2.390 of the NRC's "Rules and Practices," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and its enclosure will be placed in the NRC Public Document Room (PDR) and will be accessible from the NRC Web site at <http://www.nrc.gov/NRC/ADAMS/index.html>. No reply to this letter is required.

Sincerely,

*/RA/*

Craig Gordon, Chief  
Decommissioning Branch  
Division of Nuclear Materials Safety

Docket No. 05000029

License No. DPR-03

Enclosure: NRC Region I Inspection Report No. 50-029/2004-001

cc w/encl:

R. Kacich, YAEC President

J. Kay, Principal Licensing Engineer

G. van Noordennen, Manager, Regulatory Affairs

F. Helin, Decommissioning Director

R. Walker, Department of Public Health, Commonwealth of Massachusetts

K. Smith, Yankee Rowe Community Advisory Board

Citizens Awareness Network

Commonwealth of Massachusetts, SLO Designee

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3

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**U.S. NUCLEAR REGULATORY COMMISSION**

**REGION I**

Docket No. 05000029

License No. DPR-03

Report No. 50-029/2004-001

Licensee: Yankee Atomic Electric Company (YAEC)  
580 Main Street  
Bolton, Massachusetts 01740-1398

Facility Name: Yankee Nuclear Power Station

Location: Rowe, Massachusetts

Dates: January 22, 2004 to August 5, 2004

Inspector: J. Wray, CHP, Health Physicist, DNMS  
R. Prince, Health Physicist, DNMS

Approved by: Craig Gordon, Chief, Decommissioning Branch, Region I

Enclosure

## **EXECUTIVE SUMMARY**

Yankee Facility  
NRC Inspection Report No. 50-029/2004-001

Inspections were conducted to determine whether the decommissioning activities carried out at the Yankee Rowe facility were conducted safely and in accordance with NRC requirements. Areas reviewed included the radiation protection program, radioactive waste shipping program, quality assurance and self-assessment program, security, maintenance and operations department surveillance programs, and site organization changes.

### **Operations and Decommissioning Status**

The licensee's safety oversight, decommissioning project management, and demolition contractor organizations were sufficiently staffed and qualified to support ongoing decontamination, demolition, decommissioning and spent fuel storage activities. The inspector verified that financial assurance was current to complete decommissioning activities.

An effective maintenance and surveillance program relative to equipment and systems required for safe storage, maintenance and control of spent fuel was maintained.

The licensee maintained an adequate program for demolition of site buildings and structures.

The licensee maintained an adequate program to survey and monitor the Independent Spent Fuel Storage Installation (ISFSI) and stored spent fuel.

The licensee and their contractor, DEMCO, maintained effective corrective action programs and performed thorough audits and assessments to help self-identify and correct issues and problems.

### **Plant Support and Radiological Controls**

The licensee maintained adequate controls to limit exposures of workers to external sources of radiation. Pre-job briefings were thorough and adequately covered radiological conditions associated with a task. Posting and labeling of radioactive materials and radiation areas met regulatory requirements. Effluent monitors and personnel and equipment contamination detection monitors were properly maintained and calibrated.

Although the licensee generally maintained an effective radioactive material shipping program, two non-cited violations were identified with shipments of building and construction debris which did not meet NRC requirements.

**TABLE OF CONTENTS**

EXECUTIVE SUMMARY ..... ii

**TABLE OF CONTENTS** ..... iii

REPORT DETAILS ..... 1

Summary of Facility Activities ..... 1

I. Operations and Decommissioning Status ..... 1

O1 Conduct of Operations ..... 1

    O1.1 Organization, Management, and Cost Controls ..... 1

    O1.2 Maintenance and Surveillance Program ..... 2

    O1.3 Status of Structures and Building Demolition ..... 3

    O1.4 Independent Spent Fuel Storage Installation Operations ..... 3

    O1.5 Quality Assurance Audits and Self Assessments ..... 4

II. Plant Support and Radiological Controls ..... 5

R1 Radiological Protection Controls ..... 5

    R1.1 Occupational Exposure Controls ..... 5

    R1.2 Radioactive Waste Management and Transportation ..... 7

III. MANAGEMENT MEETINGS ..... 9

X1 Exit Meeting Summary ..... 9

X2 Other Meetings ..... 9

PARTIAL LIST OF PERSONS CONTACTED ..... A-1

LIST OF ACRONYMS ..... A-2

INSPECTION PROCEDURES USED ..... A-3

ITEMS OPENED, CLOSED, AND DISCUSSED ..... A-3

## REPORT DETAILS

### Summary of Facility Activities

Decommissioning activities at the Yankee Rowe Nuclear Power Station continued under the approval granted through a letter from the NRC to Mr. James Kay (October 28, 1996).

### **I. Operations and Decommissioning Status**

#### **O1 Conduct of Operations**

##### O1.1 Organization, Management, and Cost Controls

###### a. Inspection Scope (36801)

The inspector reviewed the licensee's organization changes and staffing to determine whether the licensee and contractor organization, staffing, and qualifications were in accordance with regulatory requirements.

###### b. Observations

The licensee announced changes in site management during this inspection period. Mr. Frank Helin was named Yankee Rowe Decommissioning Director to lead the site decommissioning project. Reporting to him are a number of departments with realigned responsibilities to better focus worker activities on demolition and site closure activities. The Nuclear Safety and Licensing, Site Closure, and Regulatory Affairs Departments now report to the Vice President - Decommissioning. The inspector discussed responsibilities and expectations with each newly appointed manager. A review of their qualifications, including educational background and work history, was performed. The inspector concluded that these managerial changes were made in accordance with regulatory requirements.

The inspector reviewed training and qualification records for a number of randomly selected site personnel from various licensee and contractor departments. Annual refresher training was verified where applicable. Qualifications of the newly assigned Radiation Protection and Chemistry Manager was reviewed. The inspector verified that he possessed the requisite experience and education for the position. No safety concerns were identified.

The inspector discussed efforts to ensure adequate funds are available to complete decommissioning. The licensee stated that a previous request to increase the decommissioning fund was approved by the appropriate regulator. This ensures the long term decommissioning and spent fuel storage needs will be met through 2022. The inspector reviewed appropriate documentation and concluded that cost estimates correlate to expenses and that adequate funds were appropriated to complete decommissioning activities. No safety concerns were identified.

Enclosure

c. Conclusions

The licensee's safety oversight, decommissioning project management, and demolition contractor organizations were sufficiently staffed and qualified to support ongoing decontamination, demolition, decommissioning and spent fuel storage activities. The inspector verified that adequate financial assurance was in place to allow decommissioning activities to be completed.

O1.2 Maintenance and Surveillance Program

a. Inspection Scope (62801)

The inspector reviewed the licensee's maintenance and surveillance program including planned and completed maintenance and surveillance activities of structures, systems and components important to the safe storage of spent fuel. The inspector reviewed the preventive and corrective maintenance program.

b. Observations and Findings

Structures, systems, and components were in good material condition. Informational tags on equipment were appropriate and housekeeping was adequate. Appropriate security and fire protection measures were in place.

The inspector reviewed the preventive maintenance and surveillance program including work prioritization, specific implementing procedures, and the work order tracking system procedure. Work is planned and coordinated with the appropriate departments in a timely manner, and work orders and specific procedures are provided in advance. The licensee effectively managed the preventive maintenance program, including the backlog and emergent work. The inspector reviewed the maintenance department surveillance schedule and the ISFSI maintenance and the ISFSI work request logs. The inspector reviewed the completed monthly surveillances of the security diesel generator and the Vertical Concrete Casks (VCC) temperature monitoring rack calibration. The inspector also reviewed the annual inspection results of the ISFSI system, structure, and component. The inspection was comprehensive and contained detailed findings which were documented appropriately. No safety concerns were identified.

c. Conclusions

An effective maintenance and surveillance program relative to equipment and systems required for safe storage, maintenance and control of spent fuel was maintained.

### O1.3 Status of Structures and Building Demolition

#### a. Inspection Scope (71801)

The inspector reviewed activities associated with building demolition and dismantlement of onsite structures. Information was gathered through a review of procedures and documents, tours of the site, and interviews with cognizant personnel.

#### b. Observations and Findings

During this inspection period, a number of significant site structures were dismantled and shipped off site for disposal. These included the Radiologically Controlled Area (RCA) Warehouse, the Waste Disposal building, the Safe Shutdown building, the Primary Water Storage Tank, the Potentially Contaminated Area (PCA) Warehouse, and the Fuel Transfer Enclosure and Yard Crane. Asbestos abatement and hazardous paint removal continued in the Primary Auxiliary Building (PAB) in preparation for final dismantlement. Dismantlement of the spherical-shaped Vapor Containment (VC) began with cutting and removal of the steel plates. Demolition activities were generally conducted in a safe manner.

During dismantlement activities, on June 6, 2004, the inspector noted three large metal plate sections of the VC sphere had toppled inward onto the bioshield. The licensee immediately stopped work and placed the demolition equipment in a safe condition. A root cause was initiated and a recovery plan was developed. The licensee determined that a change in the location of the crane used to lower the 5,000-10,000 pound sections to the ground resulted in insufficient number of pre-cuts, thereby reducing support to the three metal plate sections while adjacent sections were removed. The inspector verified that the licensee complied with industry standards and requirements for demolition, and found adequate management review of the recovery plan prior to authorizing work restart. The recovery plan was successfully executed and the inspector observed the safe removal of the metal sections. No violations were identified.

#### c. Conclusion

The licensee maintained an adequate program for demolition of site buildings and structures.

### O1.4 Independent Spent Fuel Storage Installation Operations

#### a. Inspection Scope (60855)

The inspector toured the ISFSI, reviewed records of Technical Specification (TS) required surveillances, and discussed controls of ISFSI activities with licensee operations and security personnel.

b. Observations and Findings

The inspector verified that the ISFSI pad and the VCCs in storage on the pad were in good material condition and that all radiological postings were visible and in good condition. The inspector randomly selected operator log sheets from all shifts and verified that routine surveillances on the spent fuel in dry cask storage were performed in accordance with TS requirements and that appropriate data for the VCC Thermal Monitoring Program (TS 5.4.1) were collected and logged in the control room. Roles and responsibilities for the maintenance of the ISFSI were discussed with day and night shift control room Certified Fuel Handlers and security personnel. No safety concerns were identified.

c. Conclusions

The licensee maintained an adequate program to survey and monitor the ISFSI and stored spent fuel.

O1.5 Quality Assurance Audits and Self Assessments

a. Inspection Scope (40801)

A review was performed to evaluate the effectiveness of licensee controls in identifying, resolving, and preventing issues that degrade safety or the quality of decommissioning. The inspector evaluated the licensee's self-assessment, auditing, corrective actions, and root cause evaluations through a review of licensee documents and interviews with licensee personnel. Qualifications of personnel performing audits of licensee activities were examined.

b. Observations

The inspector reviewed selected Quality Assurance Audits and Quality Surveillance Reports of licensee activities :

Audit No. Y-04-A-03-01	Training & Qualification / Security / Fitness-for-Duty / Corrective Action / Emergency Preparedness
QSR-04-102-YR	Assessment of the License Termination Plan (LTP) Project
QSR-04-103-YR	Surveillance at Radiation Safety & Control Services, INC
QSR-04-104-YR	Radioactive Waste Shipping Documentation Package
QSR-04-105-YR	ISFSI Surveillance, Calibration and Inspection
QSR-04-106-YR	Evaluation of Radiation Protection Practices
QSR-04-109-YR	Licensed Material Shipments
QSR-04-112-YR	Tours of the Restricted Area
QSR-04-113-YR	General Site Walk Down - OSHA Compliance

Audits performed by the licensee were thorough and detailed, with adequate management attention to effect timely resolution of issues.

The inspector reviewed selected field surveillances completed by the DEMCO Site Quality Assurance organization:

Enclosure

459-015	Task Package 007 Service Building Demolition (6/16/04)
459-021	Hazardous Waste Shipping (2/11/04)
459-033	Vapor Containment (VC) Dismantlement Activities (4/6/04)
459-034	Spent Fuel Sampling Activities (5/6/04)
459-035	DEMCO Training Program (6/2/04)

The field surveillances were comprehensive and showed adequate depth of knowledge of the areas inspected. No safety concerns were identified.

The inspector discussed with the licensee their program for self-identifying problems and reviewed selected Condition Reports (CR). When warranted, CRs were written and entered into the licensee's corrective action program to prevent recurrence of any potential related events. Approximately 350 CRs were initiated during this inspection period. The threshold for identifying problems and initiating a CR appears to be at an appropriate level. The inspector noted that of the approximately 350 CRs written during this inspection period, 24 were designated level 1 priority requiring a root cause determination and resolution by the Management Review Team (MRT). Fourteen of the 24 Level 1 CRs needing root cause analyses were associated with inadequate industrial safety practices (either falling debris, restraint equipment failure, or medical responses). The inspector discussed the industrial safety CRs with licensee management who stated they also identified a similar trend regarding poor industrial safety practices, and were actively working toward improving industrial safety onsite. The inspector reviewed the root cause analyses presented to the MRT. No additional safety concerns were identified.

c. Conclusions

The licensee and their contractor DEMCO maintained effective corrective action programs and performed thorough audits and assessments to help self-identify and correct issues and problems.

## **II. Plant Support and Radiological Controls**

### **R1 Radiological Protection Controls**

#### R1.1 Occupational Exposure Controls

##### a. Inspection Scope (83750)

The inspector reviewed the licensee's program to determine the capability to monitor and control radiation exposure to employees and to determine the adequacy of the licensee's radiation

protection program. The inspection consisted of observations and inspection of radiological postings, review of selected records, and interviews with cognizant personnel.

b. Observations

The inspector observed that areas of the RCA were appropriately posted and labeled for radioactive material. The inspector noted that Radiation Work Permits and RCA survey maps were posted and readily visible at the entrance to the RCA. Radiological survey maps adequately reflected radiological conditions in the posted areas. The inspector observed the monitoring of personnel and equipment upon exit from the RCA. No safety concerns were identified.

The inspector observed the morning status meeting attended by licensee and contractor personnel. Safety topics and the project status were adequately covered. The inspector noted that individual work groups involved in radiological work activities met after the general status update was completed. During these briefings a representative from the Radiation Protection group adequately covered radiological aspects of specific tasks.

The inspector reviewed exposure records for 2003. The inspector noted that no employee received more than 2 rem of exposure in calendar year 2003 and only two workers received more than 1 rem of exposure. Dose extension authorizations were completed as required. The original dose goal for 2003 was 14.8 person-rem. Changes in work scope resulted in a site exposure for 2003 of 30.939 person-rem. Licensee representatives indicated that the increase exposure was due to completion of spent fuel and Greater Than Class C (GTCC) canister transfers to the ISFSI, cleanup and decontamination of the Spent Fuel Pool, and increased demolition activity. Licensee management were kept apprised of the effect on site collective exposure associated with each task. Based on the radiological work activities performed, the inspector did not identify any concerns or safety issues relating to worker dose totals for 2003.

The inspector verified that the licensee submitted the Regulatory Guide 1.16 Radiation Exposure Summary Report for 2003 to NRC in accordance with the requirement of the licensee's Defueled Technical Specification 6.8.1 prior to the March 1, 2004, deadline. The inspector also verified that the NRC Form 5 report of onsite radiation exposures, was submitted to NRC in accordance with the requirements of 10 CFR 20.2206 (b) prior to April 30, 2004. No safety concerns were identified.

Calibration records for the liquid effluent monitor, the primary vent stack monitor, and personnel and equipment contamination monitors were reviewed. The inspector noted that the calibration of these monitors was performed in accordance with applicable procedures. The inspector observed radiation protection personnel perform the daily response and operability checks for the personnel contamination and tool contamination monitors located at the RCA exit. These response checks were performed in accordance with applicable procedures, and the inspector noted that personnel were knowledgeable of the procedural requirements and their responsibilities. The inspector reviewed selected instrument response check logs and instrument issue logs. Effluent monitors and equipment and personnel contamination monitors were noted to be calibrated and operable.

c. Conclusions

The licensee maintained adequate controls to limit exposures of workers to external sources of radiation. Pre-job briefings were thorough and adequately covered radiological conditions associated with a task. Posting and labeling of radioactive materials and radiation areas met regulatory requirements. Effluent monitors and personnel and equipment contamination detection monitors were properly maintained and calibrated.

R1.2 Radioactive Waste Management and Transportation

a. Inspection Scope (86750)

To determine compliance with NRC and transportation regulations, the inspector reviewed selected records of recent radioactive waste shipments and records for packages that were prepared and ready for shipment. The inspection consisted of field observations, interviews with cognizant personnel and review of licensee procedures relating to the packaging of radioactive waste materials for transportation.

b. Observations

Numerous intermodal containers were present throughout the RCA area and within designated storage locations of the owner controlled area. These containers were loaded with radioactive material, primarily low activity building debris, and were ready for shipment offsite. The inspector noted that the labels on these containers were readily visible and specified container radiation levels and contents in accordance with regulatory requirements. The inspector noted that the container storage locations were adequately posted and the structural integrity of the containers intact.

The inspector reviewed selected radioactive material shipping manifests. These shipments included low activity, intermodal containerized radioactive waste shipments. The inspector reviewed package dose rate survey data, radioactive material labeling, total activity, nuclide characterization, hazard waste classification, and 10 CFR 61 documentation. Documentation was in compliance with appropriate regulatory requirements.

The inspector interviewed both licensee and contractor personnel responsible for the radioactive waste management and transportation program. Personnel were knowledgeable of their responsibilities and associated regulatory and site acceptance criteria requirements.

The inspector observed the loading of non-radioactive debris onto trucks from the turbine building demolition work, and radiation protection personnel performing confirmatory surveys prior to the vehicles leaving the Protected Area. The inspector interviewed RP technicians and supervisors and confirmed that personnel were knowledgeable of the appropriate actions in the event that radioactive material was detected. No safety concerns were identified.

Two issues were inspected regarding the licensee's shipments of radioactive waste. The inspector followed up on the transportation incident which occurred on March 2, 2004, and reported to the NRC that same day. A flat bed shipment with an intermodal container loaded with contaminated building debris lost its load while traveling on a public road near the plant. The intermodal contained debris from the Safe Shutdown Building and was heading for Palmer, Massachusetts, as the first stop en route to the Envirocare Radioactive Waste disposal facility (Envirocare) in Utah. The majority of the container's contents spilled on the roadside down an embankment. The licensee immediately responded and restricted access to the affected area. Local officials and Massachusetts Department of Environmental Protection (MDEP) personnel also responded. The licensee retrieved the debris and returned it to the site within 24 hours. Environmental samples and radiation surveys were taken by the licensee and MDEP personnel following the cleanup. There was no loss of radioactive material identified, and no injuries were reported. The licensee determined that the accident occurred when the chain restraint system failed, resulting in the intermodal sliding off the flat bed of the truck.

The inspector reviewed the root cause analysis completed by the licensee and discussed the event with licensee management. Immediate corrective actions were taken, including suspension of all site waste shipments, until resolution of the problems associated with the accident were completed. The inspector verified that the licensee changed the method of securing a load to the transport vehicle from a chain arrangement to use of a pin trailer. Shipments were resumed and there were no recurring transportation incidents noted during the inspection period.

10 CFR 71.5(a) states, in part, that each licensee who transports licensed material outside their site on public highways shall comply with the applicable requirements of the Department of Transportation (DOT) regulations. DOT requirement 49 CFR 393.102 (a) states that a load must be secured during normal transport conditions and restraints must be sufficient to prevent movement forward, rearward, and laterally. The Working Load Limits of the restraining device must be at least one half of the weight of the component being secured. Using an inadequate restraining system to secure an intermodal container on the flat bed trailer is a violation of the DOT requirements contained in 10 CFR 71.5(a). However, because of the low safety significance and the timeliness and effectiveness of the licensee's corrective actions, this violation is being treated as an NCV, consistent with Section VI.A of the NRC Enforcement Policy (**NCV 50-029/04-01-01**).

The second issue occurred on March 24, 2004. The licensee was notified by Envirocare that an intermodal containing demolition debris from the Service Building was received and found to contain water in an amount greater than 1% of the container volume. This exceeds the burial site acceptance limit for free standing liquid. Although not a reportable event to the NRC, the licensee informed Region 1 of this issue the same day. The licensee immediately initiated a Condition Report (CR 04-218) and entered it into its corrective action program. A root cause evaluation was initiated. The inspector reviewed the root cause and corrective actions. The intermodal was loaded with a known amount of frozen liquid and absorbent material was added to the container with the expectation that free standing liquid would be limited to an amount less than 1% of the container volume. However, it was determined that insufficient absorbent was added because no evaluation had been made to determine the required amount. The licensee inspected all remaining containers on site and added absorbent material if free standing water

was found. Shipments en route to Envirocare which were loaded prior to March 15, 2004, were returned to the site for evaluation. The inspector verified that additional training and procedural enhancements were completed to prevent recurrence of this problem.

10 CFR 61.56(a)(3) states that solid waste containing liquid shall contain as little free standing and noncorrosive liquid as is reasonably achievable, but in no case shall the liquid exceed 1% of the volume. It was determined that shipment of an intermodal container with water in excess of 1% of the container volume is a violation of 10 CFR 61.56(a)(3). However, because of the low safety significance and the timeliness and effectiveness of the licensee's corrective actions, this violation is being treated as an NCV, consistent with Section VI.A of the NRC Enforcement Policy **(NCV 50-029/04-01-02)**.

c. Conclusions

The licensee generally maintained an effective radioactive material shipping program in compliance with regulatory requirements. However, two non-cited violations were identified which related to transportation of containers of radioactive material. One shipment did not meet DOT requirements as required by 10 CFR 71.5(a), and a second shipment did not fully satisfy the radioactive waste characterization criteria of 10 CFR 61.56 (a)(3).

### **III. MANAGEMENT MEETINGS**

#### **X1 Exit Meeting Summary**

The inspector presented the inspection results to members of licensee management periodically during the inspection, and during an exit meeting with Mr. G. Maret and others on August 5, 2004. The licensee acknowledged the findings presented by the inspector. The inspector reviewed with the licensee whether any materials examined during the inspection should be considered proprietary. While proprietary information was reviewed during the inspection, no proprietary information is contained in this report.

#### **X2 Other Meetings**

On April 22, 2004, Commissioner Merrifield toured the site and met with Bruce Kenyon, Chief, Executive Officer of YAEC, Richard Kacich, YAEC President, and other licensee representatives. Accompanying the Commissioner was Dr. Ronald Bellamy, Decommissioning Branch Chief, Region I.

On June 24, 2004, a public meeting was held at the Mohawk Trail Regional High School in Shelbourne Falls, Massachusetts to discuss the licensee's License Termination Plan. The meeting was sponsored by the NRC's Division of Waste Management, NMSS. Presentations were made by the licensee and NRC headquarters and regional staff. NRC responded to questions from the public following the meeting. Approximately 50 people were in attendance including representatives from local television and print media and Citizen Against Nuclear Power.

**PARTIAL LIST OF PERSONS CONTACTED**

- G. Babineau, YAEC, Safety Oversight Manager
- \*W. Blackadar, YAEC, Radiation Protection Oversight
- \*D. Calsyn, YAEC, Quality Assurance Manager
- \*T. Carmody, Assistant Project Director, DEMCO
- \*R. Dee, YAEC, Scheduler
- \*S. Garvie, YAEC, Security Supervisor
- \*E. Heath, YAEC, Asst. Safety Oversight Manager
- R. Kacich, President, YAEC
- J. Kay, YAEC-Regulatory Affairs
- K. LaDuke, YAEC, QA Auditor
- \*G. Maret, Vice President of Decommissioning, YAEC
- \*D. Maffai, Radwaste Manager, Duratek
- K. Myers, Health Physics Manager, Duratek
- \*D. Montt, HP and Chemistry Supervisor, YAEC
- \*M. Niehoff, Project Director, DEMCO
- \*C. Palmer, Radiological Engineer, Duratek
- \*N. Rademacher, YAEC Decommissioning Project Manager
- S. Racz, YAEC, Sr. Technical Specialist-QA
- \*K. Shea, YAEC Plant Maintenance Engineer
- \*D. Smith, YAEC FSS Project Manager
- K. Smith, YAEC, Communications Manager
- \*M. Vandale, YAEC Oversight
- \*F. Williams, YAEC, Plant Superintendent
- B. Wood, YAEC, Site Manager (former)
- M. Zurlo, QA Manager, DEMCO

\* These individuals participated in the exit briefing held on August 5, 2004.

**LIST OF ACRONYMS**

ADAMS	Agency Document Access and Management System
ALARA	As Low As Reasonably Achievable
ASW	Auxiliary Service Water
CAB	Community Advisory Board
CFR	Code of Federal Regulations
CR	Condition Report
DCR	Design Change Request
EP	Emergency Preparedness
GTCC	Greater Than Class C
HDLF	High Dirt Load Filter
ICM	Interim Compensatory Measures
ISFSI	Independent Spent Fuel Storage Installation
LTP	License Termination Plan
MDEP	Massachusetts Department of Environmental Protection
MRT	Management Review Team
NCV	Non-cited Violation
NPDES	National Pollutant Discharge Elimination System
ODCM	Offsite Dose Calculations Manual
PAB	Primary Auxiliary Building
PCA	Potentially Contaminated Area
PDR	Public Document Room
QA	Quality Assurance
QSR	Quality Surveillance Reports
RCA	Radiological Controlled Area
RP	Radiation Protection
SFP	Spent Fuel Pool
TI	Temporary Instruction
TS	Technical Specification
VC	Vapor Containment
VCC	Vertical Concrete Cask
VEMA	Vermont Emergency Management Agency
YAEC	Yankee Atomic Electric Company
YNPS	Yankee Nuclear Power Station

**INSPECTION PROCEDURES USED**

IP 36801	Organization, Management, and Cost Controls
IP 40801	Self Assessment, Auditing, and Corrective Action
IP 60855	Operation of ISFSI
IP 62801	Maintenance and Surveillance
IP 71801	Decommissioning Performance and Review Status
IP 83750	Occupational Radiation Exposure Control
IP 84750	Radioactive Waste Treatment and Effluent & Environmental Monitoring
IP 86750	Radwaste Management and Transportation

**ITEMS OPENED, CLOSED, AND DISCUSSED**

Discussed

2004-001-01	NCV	Failure to comply with 10 CFR 71.5 (a) - vehicle load restraint
2004-001-02	NCV	Exceeding Radwaste Characterization Acceptance Criteria - excessive water, 10 CFR 61.56(a) (3).

Opened

NONE

Closed

NONE