January 14, 1999 SECY 99-015

For: The Commissioners

From: James L. Blaha, Assistant for Operations, Office of the EDO ***/RA/***

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING JANUARY 8, 1999

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*No input this week.

James L. Blaha
Assistant for Operations, OEDO

Contact:
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Office of Nuclear Reactor Regulation
Items of Interest
Week Ending January 8, 1999

Technical Assistance on Digital Instrumentation and Control Systems to the Mexican Nuclear Regulatory Authority

During the week of December 14, 1998, Jerry Mauck, Section Chief, Instrumentation and Controls Branch, visited the Mexican nuclear regulatory authority (CNSNS) offices in Mexico City and the Laguna Verde Nuclear Power Plant site in Vera Cruz as part of an IAEA assistance project to provide technical support in the area of digital instrumentation and control (I&C) systems. Mr. Mauck provided presentations and discussions in the areas of software quality assurance, configuration management, use of commercial-off-the-shelf software, the Year 2000 problem, oscillation power range monitoring system for BWRs, and digital system operating experience. The information provided was based on the NRC guidance on acceptance of digital I&C systems contained in Chapter 7 of the Standard Review Plan, NUREG-0800. In addition, the Laguna Verde licensee, CFE, presented information on their Year 2000 readiness program and on digital system operating problems and deficiencies identified at Laguna Verde. Information exchanges with CNSNS on this subject will continue in the future.

Participation by Telephone in Programs on the Year 2000 (Y2K) Problem in Nuclear Power Plants

On January 4, 1999, Jared Wermiel, Matthew Chiramal, and William Dean, NRR and Beth Hayden, OPA participated by telephone in a radio call-in program sponsored by a local radio station in Waterford, CT on the impact of the Y2K problem in nuclear power plants and the NRC efforts at addressing the problem. Also participating in the program were representatives from Northeast Utilities to speak on the Y2K readiness program at Millstone, Susan Perry Luxton representing a local public citizens group, and Paul Gunther of the Nuclear Information and Resources Service. Mr. Wermiel provided information for listeners on the actions taken by the NRC with its licensees on the Y2K problem and answered questions and addressed issues raised by Ms. Luxton, Mr. Gunther and members of the public. Mr. Wermiel and Ms. Hayden also participated by phone in a followup cable TV program on the same topic sponsored by Ms. Luxton where additional information on the Y2K problem and NRC efforts to address it were discussed.

Status of Inspection and Replacement of Baffle-Former Bolt Activities at Point Beach Unit 2

In December, the Wisconsin Electric Power Company began a voluntary program at Point Beach Unit 2 to inspect, replace and test baffle-former bolts, located in the reactor internals, under the provisions of 10 CFR 50.59. The inspection effort was looking for cracking due to a stress-corrosion type of a mechanism, as discussed by IN 98-11, “Cracking of Reactor Vessel Internal Baffle Former Bolts in Foreign Plants.” Inspection of the 728 baffle-former bolts at Point Beach revealed apparent cracking in 24 or 3.3% of the bolts, consistent with the experience at foreign PWRs reported in IN 98-11. This finding is not consistent with the results of inspections at Farley Unit 1 (described in the Weekly Information Report of December 11), where inspection of 1086 bolts revealed no indications of apparent cracking. The differences in the findings from Point Beach and Farley are thought to be attributable to differences in configuration (upflow vs.

downflow of baffle/barrel region flow), plant operation, and baffle-former bolt material used in the plants.

The bolt replacement activities are focused on replacing a portion of the bolts (~40%) to a specific pattern determined by Westinghouse to provide sufficient margins against potential accident conditions, with all of the non-replaced bolts assumed to support no load in the evaluation.

Metallurgical examination of bolts removed from both Point Beach Unit 2 and Farley Unit 1 will be performed in the coming year to provide understanding of the cracking mechanism(s) and controlling factors, such as neutron fluence, material composition, water chemistry, and applied loading. It is expected that the Westinghouse Owners Group (WOG) will provide the staff with their assessment of the plant findings at a future meeting between the WOG and NRC staff.

Additional plants expected to inspect (and potentially replace) baffle-former bolts in the next year include Ginna (similar to Point Beach Unit 2) and Farley Unit 2 (similar to Farley Unit 1).

Approval of ANO2 Risk-Informed Inservice Inspection Pilot Program

By letter to Entergy Operations, Inc. dated December 29, 1998, the NRC approved on schedule the risk-informed inservice inspection (RI-ISI) program requested by the Arkansas Nuclear One, Unit 2, (ANO2) licensee as an alternative of current inservice requirements of ASME Code, Section XI under 10 CFR 50.55a(a)(3)(i).

The alternative is based on a risk-informed process described in Code Case N-578 “Risk-Informed Requirements for Class 1, 2 and 3 piping - Method B” and developed in accordance with the more detailed provisions described in the Electric Power Research Institute (EPRI) Technical Report No. TR-106706, Risk Informed Inspection Evaluation Procedure, Interim Report,” as augmented by ANO2 plant-specific methodologies. The licensee specifically, proposed to redistribute the piping inspections to focus on high-risk piping segments of safety-related piping systems while minimizing the number of inspections performed on low-risk systems and piping segments. In addition, more volumetric examinations were considered over surface exams for higher risk piping segments. The result of this redistribution was an overall reduction in the number of piping examinations performed from the original value of 336 examinations to the 167 examinations that were proposed by the licensee while achieving a small reduction in risk. Since ANO2 is the first plant using the RI-ISI methodology described in the EPRI report, as augmented by ANO2 plant-specific methodologies, the program is considered as the pilot program for the EPRI methodology.

The completion of ANO2 Safety Evaluation (SE) on RI-ISI is item B.6 on the Chairman’s Tasking Memorandum and was scheduled to be completed on December 31, 1998. The staff issued the SE on December 29, 1998.

Fire Protection Functional Inspection Program

On November 10, 1998, the Office of Nuclear Reactor Regulation held a workshop on reactor fire protection inspections. During the workshop the Nuclear Energy Institute (NEI) offered a possible industry initiative that would involve licensee-managed fire protection program self-

assessments as an alternative to continued NRC fire protection functional inspections (FPFIs). The self-assessments would be managed and conducted by the licensees, would be based on inspections “modules” derived from the existing FPFI procedure, and would be performed over time at all plants. Under this proposal, NRC oversight would consist of limited NRC inspections of selected licensee self-assessments. On December 14, 1998, the staff met with NEI to learn more about its proposal. During the meeting, NEI offered a lesser proposal. Specifically, while NEI would develop the inspection modules, it would only make them available to the licensees for use on a voluntary basis. It would not implement a formal industry initiative to conduct fire protection program self-assessments at all plants. NEI also suggested that the NRC role should be established solely by the new NRC inspection and performance assessment processes that are currently being developed. The staff questioned whether or not the revised NEI proposal was responsive to the lessons learned from the FPFI pilot program. NEI said that it believed that its revised proposal was adequate in light of the new performance assessment process and the results of the pilot FPFIs. NEI will formalize its proposal for fire protection program inspections in a letter by January 20, 1999.

The Probabilistic Safety Assessment Branch/Technical Specifications Branch Meeting With Industry Representatives on Risk-Informing the Standard Technical Specifications (RI-ITS), Region II, December 8, 1998

The Probabilistic Safety Assessment Branch representatives joined with the Technical Specifications Branch representatives at the above meeting on risk-informing the standard technical specifications (RI-ITS). All participants (industry and staff) agreed that RI-ITS is “the right thing to do.” It was decided that, initially, improvements in the current 10 CFR 50.36 should be explored. Industry representatives expressed concern with the current ITS especially in the area of required actions. Additionally, the staff is considering 1) re-focusing surveillance requirements (SR) on important safety functions, 2) optimizing and moving surveillance test intervals (SAIS) to the licensee maintenance rule program, 3) redefining allowed outage times (AOTS) in terms of a “backstop” value and application of the licensee configuration risk management program (CAMP), 4) easy generic improvements -- AOTS missed surveillance requirements; mode restraints, 5) clarification of functionality vs. operability, and 6) a long-term goal of fully risk-informed, performance-based ITS.

Comanche Peak Charging Pump at Extension

On December 30, 1998, the staff issued the safety evaluation report for the proposed charging pump allowed outage time extension from 72 hours to 7 days. In reviewing the risk associated with the change, the Probabilistic Safety Assessment Branch used a three-tiered approach, consistent with Regulatory Guide (RG) 1.177. The staff concluded that the probabilistic risk assessment (PRA) used in support of the change was of sufficient quality, scope and level of detail for the proposed application, and the risk impact of the change would be small. The staff also found that the licensee had an adequate control on potential risk significant configurations, and implemented a configuration risk management program which is consistent with the regulatory guide. Therefore, the staff concluded that PRA insights and findings supported the proposed change.

Sequoyah Emergency Diesel Generator Allowed Outage Time Extension

On December 16, 1998, the staff issued the safety evaluation report for the proposed emergency diesel generator allowed outage time extension from 72 hours to 7 days. In reviewing the risk associated with the change, the Probabilistic Safety Assessment Branch used a three-tiered approach, consistent with Regulatory Guide (RG) 1.177. The staff concluded that the probabilistic risk assessment (PRA) used in support of the change was of sufficient quality, scope and level of detail for the proposed application, and the risk impact of the change would be small. The staff also found that the licensee had an adequate control on potential risk significant configurations, and implemented a configuration risk management program which is consistent with the RG guidance. Therefore, the staff concluded that PRA insights and findings supported the proposed change.

Tritium Production

On December 22, 1998, Department of Energy Secretary Bill Richardson made a decision regarding future production of tritium for the United States nuclear weapon stockpile. The United States has not produced tritium since the 1988 closure of the Savannah River facility. DOE is responsible for establishing the capability to produce tritium by the end of 2005, in accordance with a Presidential Decision Directive. The three main options pursued by DOE were construction of an accelerator at Savannah River, completion of Bellefonte Unit 1, and use of Watts Bar and/or Sequoyah reactor facilities. Secretary Richardson selected the Watts Bar/Sequoyah option on the basis that it would require very little capital expenditure, and DOE will pay only for tritium production as needed. TVA also agreed to provide irradiation services under the Economy Act, which requires services between different government agencies to be provided at actual cost.

Completion of Bellefonte 1, which is 88% constructed, would have required an expenditure of approximately $2 billion, and construction of an accelerator would cost considerably more. The NRR staff met with TVA on March 17, 1998, to discuss the Bellefonte option. Secretary Richardson visited the Watts Bar, Bellefonte, and Savannah River sites a number of times before making his decision.

To confirm the technology, four lead test assemblies were installed in the Watts Bar reactor during the last refueling outage (October 1997). The staff issued a license amendment to support the test and is in the process of reviewing a topical report submitted by DOE to support production core loads. A Safety Evaluation on the topical report is scheduled to be issued in March 1999. TVA has informed the staff of their intention to request approval in the form of license amendments for Watts Bar and Sequoyah in mid-1999 to produce tritium.

Quad Cities Unit 1

Before startup of Unit 1 in early December 1998, ComEd had General Electric perform a safety assessment of jet pump crack indications to support restart. The objective of the safety assessment was to perform a qualitative evaluation of the crack indications to determine the safety consequences of operation with the indications left as-is. GE compared the potential cracking scenarios for the indications with the cracking scenario in BWRVIP-28. The conclusion

of the assessment was that there is an extremely low probability of the cracking causing a safety concern; therefore, operation was acceptable from a safety perspective.

ComEd also requested GE to perform a crack growth evaluation to support continued operation. A crack length limit of 10 inches was chosen (a 1994 repair on jet pump 5/6 had a crack length of 15"). The conclusion of this evaluation was that operation at 95% of rated recirculation drive flow, which is where unit 1 is currently operating, is allowed for up to 4400 hours. This would put the need for a repair into May 1999. Subsequently, ComEd elected for a short planned outage to repair these cracks starting April 10, 1999. Vessel disassembly, re-assembly and testing will require about 9 days and in-vessel repair time would be added to this.

Point Beach

The NRC granted enforcement discretion to Wisconsin Electric for continued operation of Pt. Beach Unit 1 on January 5, 1999. The Notice of Enforcement Discretion (NOED) was granted verbally by Region III to avoid an undesirable transient caused by an unplanned plant shutdown. Pt. Beach Unit 1 entered a Technical Specification required 7 hour plant shutdown at 1240 CST due to both trains of safety injection being declared inoperable as a result of a frozen recirculation line.

The licensee requested a NOED for an additional 6 hours to allow time to establish an alternate recirculation path and declare one train of safety injection operable. The NOED was granted at approximately 1820 CST.

The original frozen line was thawed at 2234 CST on January 5. Unit 1 was ramping back up to 100% power on January 7. Operations personnel at Pt. Beach have been evaluating all freeze protection at the plant and have walked down systems, but have not identified any other problems.

Improved Tech Specs for Byron and Braidwood

On December 22, 1998, the staff issued the Improved Technical Specifications (ITS) for Commonwealth Edison's Byron and Braidwood Stations. The amendments were requested on December 13, 1996. ComEd had proposed a significant number of deviations from the standard TS, many of which were based on industry-proposed generic changes. This expanded the scope of the staff's review and resulted in 20 supplements to the original submittal (approximately 40% of the original submittal was revised). The licensee plans to implement the ITS at Byron by January 15, and at Braidwood by February 19, 1999, to support schedule improvements for the spring refueling outages. These are the 40th, 41st, 42nd, and 43rd units that have been approved to convert to the ITS.

Management Changes

Jack McElwain, who was the Vice President-Recovery for the Clinton Power Station (Illinois Power) is now the Senior Vice President and Chief Nuclear Officer replacing Walt MacFarland. Mr. McElwain is part of the management services contract with PECO Energy as was Mr. MacFarland.

Office of Nuclear Material Safety and Safeguards
Items of Interest
Week Ending January 8, 1999

Low Seal Pressure Alarm on MC-10 at Surry Power Station

Following the ice storm at the end of December 1998, Virginia Power had an indication of low seal pressure on the MC-10 cask at the Surry Power Station. Preliminary indications were that the cask seals were fine but there was a problem with the instrumentation that measures the differential pressure. A suspected leak in the seals that house the instrumentation was being investigated. The housing was resealed and re-evacuated. The alarm cleared. A sample showed no indication of helium or activity. It appears that replacement of the seals on the instrumentation has resolved the issue. The licensee is continuing to monitor seal leakage to insure that their corrective actions have resolved the issue. Region II is following up on the event, and the Spent Fuel Project Office is providing technical support as needed.

Spent Fuel Project Office Meets with Transnuclear, Inc., on the TN-32 Dry Cask Storage Design

On January 5, 1999, members of the Spent Fuel Project Office (SFPO) and Pacific Northwest National Laboratory (PNNL) met with Transnuclear, Inc., (TN) to discuss the on-going review of the TN-32 dry storage cask application. SFPO staff and PNNL presented their remaining issues in the review, and TN provided its proposed solutions to these issues. Specific details of the solutions were discussed among the respective team members. TN committed to provide its final safety analysis report revision to the staff by the last week of January 1999. The TN-32 is planned for use at McGuire Power Station and a Duke representative was present. Philadelphia Electric Company (PECO) Energy representatives were also present at the meeting.

Spent Fuel Project Office Meets with Private Fuel Storage, L.L.C.

On January 6-7, 1999, the Spent Fuel Project Office (SFPO), Office of Nuclear Material Safety and Safeguards, met with Private Fuel Storage (PFS), L.L.C., an applicant for a license to construct and operate an away-from-reactor independent spent fuel storage installation on the reservation of the Skull Valley Band of Goshute Indians. The purpose of the meeting was to discuss clarifications to the staff's two recent requests for additional information (RAIs). The first day of the meeting focused on clarification to the second round site safety review RAI. The second day focused on clarifications to the first round environmental RAI. Responses to both RAIs are scheduled to be submitted by PFS in February 1999. Also participating in the meetings were staff from other Nuclear Regulatory Commission offices (i.e., Office of the General Counsel and Office of Nuclear Reactor Regulation) that are assisting SFPO in the licensing review. In addition, the Center for Nuclear Waste Regulatory Analyses (assisting in the site safety review) and Oak Ridge National Laboratory (assisting in the preparation of the Environmental Impact Statement) participated in the meeting.

Materials Program Counterpart Meeting

On January 6-7, 1999, Division Directors from the Office of Nuclear Material Safety and Safeguards (NMSS) met with their regional counterparts, the Director, Office of Enforcement, the Deputy Director of the Office of State Programs, and the Director of the Technical Training

Division, Office for the Analysis and Evaluation of Operational Data, to discuss a series of policy and technical program issues. Discussion centered on the streamlining initiatives planned or already underway in various NMSS program areas. The meeting took place at the Nuclear Regulatory Commission Headquarters in Rockville, Maryland.

10 CFR Part 70 Rulemaking

The staff is continuing to review the comments provided by the Nuclear Energy Institute (NEI) regarding the Integrated Safety Analysis (ISA), including decommissioning ISA baseline design criteria and nuclear criticality safety. Some comments on the revised rule language that was posted on the 10 CFR Part 70 Website in December 1998 have been received “online”. A public meeting to discuss nuclear criticality safety technical comments provided by industry on the Part 70 rule and associated Standard Review Plan is scheduled for January 13-14, 1999. NEI has informed the staff that it intends to submit the balance of its comments on the rule during the week of January 11, 1999.

Office of Nuclear Regulatory Research
Items of Interest
Week Ending January 8, 1999

Assessment of Radiological Consequences for the Perry Pilot Plant Application Using the Revised (NUREG-1465) Source Term

In conjunction with ongoing NRR/RES activities in support of implementation of the revised (NUREG-1465) source term at operating reactors, Division of Systems Technology staff has worked closely with NRR to assess the Perry pilot plant application. In that regard, DST staff has assisted in evaluating the appropriateness of the licensee's treatment of both thermal hydraulic boundary conditions and fission product deposition mechanisms which influence the calculation of offsite and control room doses. In consultation with NRR, DST staff has also performed, numerous in-house sensitivity studies using the RADTRAD code to calculate radiological consequences. The results of the assessment show that the radiological consequences associated with removing the main steam isolation valve (MSIV) leakage control system and increasing allowable MSIV leakage rate, together with implementation of the revised source term, are acceptable. The detailed results of the assessment are given in the report AEB-98-03, "Assessment of Radiological Consequences for the Perry Pilot Plant Application Using the Revised (NUREG-1465) Source Term," that was forwarded to NRR by memorandum dated December 17, 1998.

The Perry plant proposed changes to its facility, including removing the MSIV leakage control system and increasing the allowable MSIV leakage rate. To support its proposed changes, the Perry plant submitted a consequence assessment using the revised source term to show that radiological consequence limits were not exceeded. This is the first license amendment being considered by the NRC based on the revised source term. The revised source term is a more realistic source term, most notable in the areas of physical form of the radionuclide release and the release timing. Because it is more representative of source terms that might occur during an accident, its application allows for burden reduction and safety enhancements in both plant design and operating procedures, as demonstrated in the generic rebaselining study (SECY-98-154, June 1998).

Office for Analysis and Evaluation of Operational Data
Items of Interest
Week Ending January 8, 1999

PRELIMINARY NOTIFICATIONS:

1. PNO-99-001, Northeast Utilities (Millstone 2, 3), UNIT 2 FUEL RELOAD AND UNIT 3 RETURN TO POWER OPERATION

2. PNO-II-98-053A, Burns Cooley Dennis, Inc., RECOVERY OF LOST PORTABLE GAUGE

3. PNO-99-001, Northern States Power Co. (Prairie Island 1), STATION AUXILIARY TRANSFORMER EXPLOSION AND FIRE

Office of Administration
Items of Interest
Week Ending January 8, 1999

Front Entrance Extension

The start of the front entrance extension project, expected to begin January 18, 1999, has been delayed to correct problems identified in the structural steel details. The construction contractor has submitted a "Request for Information" (RFI) to clarify the issue. The RFI is being responded to by the A/E design firm and a response is due the week of January 11, 1999. A new construction schedule will be prepared by the construction contractor after the RFI has been addressed.

Rulemaking Activities for the Week Ending January 8, 1999

Procedures Applicable to Proceedings for the Issuance of Licenses for the Receipt of High-Level Radioactive Waste at a Geologic Repository (Part 2)

A final rule that amends the Commission's Rules of Practice for the licensing proceeding on the disposal of high-level radioactive waste at a geologic repository was published in the Federal Register on December 30, 1998 (63 FR 71729). The final rule allows the application of technological developments that have occurred since the original rule was issued in 1989 while retaining the goal of facilitating the Commission's ability to comply with the schedule for a decision on the construction authorization established by the Nuclear Waste Policy Act. The final rule becomes effective January 29, 1999.

Chief Information Officer
Items of Interest
Week Ending January 8, 1999

Freedom of Information and Privacy Act Requests received during the 4-Day Period of January 4, 1999 through January 7, 1999:

Nuclear Fuel Services (NFS) Inc., Erwin, TN. (FOIA/PA 99-083)

Environmental assessment of property located at 1415 Park Ave., Hoboken, NJ, known as Just Parking, formerly known as United States Testing Co., or Qualitest. (FOIA/PA 99-084)

Office of Human Resources
Items of Interest
Week Ending January 8, 1999

Departures		
BOYNTON, Scott	Sr Resident Inspector	RIV
KALAMON, Michele	Secretary	RI
LEW, Patricia	General Engineer	RES
NICHOLSON, Larry	Deputy Director/DRS	RI

Office of Public Affairs
Items of Interest
Week Ending January 8, 1999

Media Interest

Chairman Jackson was interviewed by a reporter from WOL radio, and by a reporter from US Black Engineer magazine.

Hub Miller, Region I, was interviewed by the Philadelphia Inquirer for a story on an alleger at Susquehanna.

Press Releases	
Headquarters:	
99-01	Note to Editors: ACRS subcommittee meeting date and topic changed
99-02	Chairman, Vice-Chairman and Member-at-Large Elected to NRC Advisory Committee on Reactor Safeguards
99-03	Media Workshop Scheduled February 23-24
99-04	NRC Modifies Policy on Enforcement Discretion in Severe Weather Conditions
Regions:	
I-99-01	Note to Editors: January 11 meeting with Northeast Utilities on Millstone
I-99-02	NRC Names New Resident Inspector at Nine Mile Point Nuclear Plant
II-99-01	NRC Receives Request from CP&L for Use of Additional Harris Plant Spent Fuel Pools
II-99-02	NRC Names New Senior Resident Inspector at Summer Nuclear Power Plant
III-99-01	Nuclear Regulatory Commission to Discuss Findings of Inspection Into Dec. 9 Fire at Portsmouth Uranium Plant
III-99-02	NRC Names New Senior Resident Inspector at Monticello Nuclear Power Plant
III-99-03	Meeting at Portsmouth Uranium Plant Canceled Because of Severe Weather
IV-99-02	NRC Proposes to Fine Entergy $55,000 for Violation at River Bend Nuclear Plant

Office of International Programs

Items of Interest
Week Ending January 8, 1999

DOE Meeting on NEA Matters

As the new representative to the NEA Steering Committee, William Magwood, Director Office of Nuclear Energy, has scheduled a meeting on January 12, 1999, for the purpose of reviewing DOE Program of Work priorities for each of the six Technical Standing Committees. Other members of the U.S Delegation to the NEA Steering Committee (State Department, NRC) have been invited to attend. Janet Gorn, OIP, and Tom King, RES, will represent NRC.

Office of the Secretary
Items of Interest
Week Ending January 8, 1999

Documents Released to Public	Date	Subject
Decision Documents		
1. SECY-98-300	12/23/98	Options for Risk-Informed Revisions to 10 CFR Part 50 - "Domestic Licensing of Production and Utilization Facilities"
Information Papers		
1. SECY-98-291	12/18/98	Plan for Improving the Effectiveness of NRC's Training Programs
2. SECY-98-293	12/18/98	Status Report of the U.S. Nuclear Regulatory Commission Task Force on Oversight of the U.S. Department of Energy, in Response to COMSECY-96-053-DSI 2 (Report No. 6)
3. SECY-98-295	12/18/98	Effectiveness of Criteria for Funding Agreement State Training
4. SECY-98-299	12/23/98	Weekly Information Report - Week Ending December 18, 1998

Commission Correspondence

1. Letter to Gilbert F. Casellas, EEOC, dated December 30, 1998, provides the NRC's FY98 Program Plan Update and Accomplishment Report on the Affirmative Action Program Plan for Hiring, Placement, and Advancement of Individuals with Handicaps (incoming dated December 10, 1998 also released).

2 Letter to Senators Christopher Dodd and Joseph Lieberman and Representative Sam Gejdenson dated January 7, 1999 provides a follow up to Chairman Jackson's September 11, 1998 letter concerning actions on investigations completed by the Office of Investigations related to the Millstone facility (incoming dated September 1, 1998 also released).

Federal Register Notices Issued

1. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Planning and Procedures; Notice of Meeting on February 2, 1999.

2. Advisory Committee on Reactor Safeguards; Meeting Notice for February 3-6, 1999.

3. Advisory Committee on Reactor Safeguards, Subcommittee meeting on Planning and Procedures; Meeting Notice for February 2, 1999.

Region I
Items of Interest
Week Ending January 8, 1999

Maine Yankee Plant

On Wednesday, December 30, 1998, the Maine Yankee Atomic Power Station, which is currently undergoing decommissioning, went to a “cold and dark” condition. After completing modifications to the electric power supply system, the site has essentially abandoned most of the electric distribution network at the facility. Electric power is being supplied only to the Spent Fuel Building, the Control Room, the Health Physics Control Point, some temporary lighting circuits, and a few special locations (e.g. the meteorological tower). Achieving the “cold and dark” status was an important milestone in the decommissioning schedule. The de-energizing of the balance of the plant will allow decommissioning activities to commence safely in these areas without concern for interacting with electrically energized systems.

Region II
Items of Interest
Week Ending January 8, 1999

Florida Power and Light Company - St. Lucie

On January 7, 1999, the Regional Administrator and other Region II representatives held a pre-decisional enforcement conference with representatives of Florida Power and Light Company. Representatives of the Office of Nuclear Reactor Regulation also participated in the conference concerning the licensee's perspective on apparent violations relating to the licensee's fire protection program at the St. Lucie facility.

Office of Congressional Affairs
Items of Interest
Week Ending January 8, 1999

CONGRESSIONAL HEARING SCHEDULE, NO. 1					
OCA Contact	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Madden	TBA	TBA	NRC Commission	Oversight/Authorization	Sen. Inhofe/Graham Clean Air, Wetlands, Private Property, and Nuclear Safety Environment and Public Works
Combs	TBA	TBA	TBD	High Level Waste Program	Sen. Murkowski/Bingaman Energy and Natural Resources
Combs	TBA	TBA	TBD	High Level Waste Program	Rep. Barton Energy and Water Commerce