Attachment 2

Mark-up of Technical Specifications

Dominion
Surry Power Station Units 1 and 2

4. The reactor thermal power level shall not exceed 118% of rated power.

B. The safety limit is exceeded if the combination of Reactor Coolant System average temperature and thermal power level is at any time above the appropriate pressure line in TS Figures 2.1-1, 2.1-2 or 2.1-3; or the core thermal power exceeds 118% of the rated power.

In the event the Safety Limit is violated, the facility shall be placed in at least HOT SHUTDOWN within 1 hour. C1

Basis

To maintain the integrity of the fuel cladding and prevent fission product release, it is necessary to prevent overheating of the cladding under all operating conditions. This is accomplished by operating within the nucleate boiling regime of heat transfer, wherein the heat transfer coefficient is very large and the clad surface temperature is only a few degrees Fahrenheit above the reactor coolant saturation temperature. The upper boundary of the nucleate boiling regime is termed Departure From Nucleate Boiling (DNB) and at this point there is a sharp reduction of the heat transfer coefficient, which would result in high clad temperatures and the possibility of clad failure. DNB is not, however, an observable parameter during reactor operation. Therefore, DNB has been correlated to thermal power, reactor coolant temperature and reactor coolant pressure which are observable parameters. This correlation has been developed to predict the DNB flux and the location of DNB for axially

## 2.2 SAFETY LIMIT, REACTOR COOLANT SYSTEM PRESSURE

Applicability

Applies to the maximum limit on Reactor Coolant System pressure.

Objective

To maintain the integrity of the Reactor Coolant System.

Specification

A. The Reactor Coolant System pressure shall not exceed 2735 psig with fuel assemblies installed in the reactor vessel.

B. In the event the Safety Limit is violated, the facility shall be placed in at least HOT SHUTDOWN within 1 hour. $\boxed{C1}$

Basis

The Reactor Coolant System[1] serves as a barrier which prevents radionuclides contained in the reactor coolant from reaching the environment. In the event of a fuel cladding failure the Reactor Coolant System is the primary barrier against the release of fission products. The maximum transient pressure allowable in the Reactor Coolant System pressure vessel under the ASME Code, Section III is 110% of design pressure. The maximum transient pressure allowable in the Reactor Coolant System piping, valves and fittings under USAS Section B31.1 is 120% of design pressure. Thus, the safety limit of 2735 psig (110% of design pressure) has been established.[2]

6.0 ADMINISTRATIVE CONTROLS

6.1 ~~Organization, Safety and Operation Review~~ Responsibility, Organization and Qualifications

6.1.1 ~~Specification~~ Responsibility

6.1.1.1 ~~A.~~ The ~~Site Vice President~~ plant manager shall be responsible for the overall unit operation ~~of the facility~~. ~~In his absence, the Manager - Station Operations and Maintenance shall be responsible for the safe operation of the facility.~~ During ~~the~~ his absence ~~of both~~ the ~~Site Vice President~~ plant manager will delegate in writing the succession to this responsibility. C1 C3

Insert 1

6.1.2 Organization

6.1.2.1 Onsite and offsite organizations

~~B. An~~ Onsite and ~~an~~ offsite organization(s) shall be established for ~~facility~~ unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

~~1.~~ a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the ~~UFSAR~~ QA Program. C4

Insert 2

~~2.~~ b. The ~~Site Vice President~~ Plant manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant. C5

~~3.~~ c. ~~The Vice President - Nuclear Operations~~ a specified corporate officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining and providing technical support to the plant to ensure nuclear safety.

~~4.~~ d. ~~The management position responsible for training of the operating staff and the position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.~~ C6

~~5. The management position responsible for health physics shall have direct access to that onsite individual having responsibility for overall facility management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.~~

Insert 3

**Insert 1**

6.1.1.2 The Shift Manager shall be responsible for the control room command function. C3

**Insert 2**

The plant-specific titles of those personnel fulfilling the responsibilities of the positions delineated in these Technical Specifications shall be maintained in appropriate administrative documents. C4

**Insert 3**

The individuals who train the operating staff, carry out health physics, or perform quality assurance functions may report to the appropriate onsite manager; however, these individuals shall have sufficient organizational freedom to ensure independence from operating pressures. C6

6.1.2.2 Unit Staff

B. The ~~facility~~ unit staff organization shall ~~conform to~~ include the following ~~requirements~~:

6.1.3 Unit Staff Qualifications

6.1.3.1 1. Each member of the ~~facility~~ unit staff shall meet or exceed the minimum qualifications ~~of ANS 3.1 (12/79 Draft)* for comparable positions, except for:~~ as specified in the Quality Assurance Program. C7

~~a. The Superintendent - Radiological Protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.~~ C8

6.1.3.2 Insert 4

6.1.2.2.d b. The ~~Superintendent~~ Manager Operations shall hold (or have previously held) a Senior Reactor Operator License for Surry Power Station or a similar design Pressurized Water Reactor plant. C5

c. The Supervisor Nuclear Shift Operations shall hold an active Senior Reactor Operator License for Surry Power Station. C9

2. Incumbents in the positions of Shift ~~Supervisor~~ Manager, ~~Assistant Shift~~ Unit Supervisor (SRO), Control Room Operator ~~- Nuclear~~ (RO), and ~~Shift Technical Advisor~~ the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4). C9 C5

~~3. The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the licensed facility staff meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4). Also, a retraining and replacement training program for non-licensed facility staff shall meet or exceed the recommendations of Section 5 of ANS 3.1 (12/79 Draft)*.~~ C7

6.1.2.2.a 4. Each on-duty shift shall be composed of at least the minimum shift crew composition for each unit as shown in Table 6.1-1.

6.1.2.2.b 5. A ~~health physics~~ radiation protection technician shall be on site when fuel is in the reactor. Insert 5 C5 C10

6.1.2.2.c 6. All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator limited to fuel handling who has no other concurrent responsibilities during this operation.

~~* Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operational Phase."~~ C7

## Insert 4

For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator and a licensed Reactor Operator are those individuals who, in addition to meeting the requirements of TS 6.1.3.1 perform the functions described in 10 CFR 50.54(m). C8

## Insert 5

The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the position. C10

~~7. Deleted.~~

~~8. Deleted.~~

9. ~~The health physics technician requirement of Specification 6.1.B.5 may not be met for a period of time not to exceed 2 hours in order to accommodate an unexpected absence provided immediate action is taken to fill the required position.~~ C10

5.2.2.e 10. Procedures will be established to insure that NRC policy statement guidelines regarding working hours established for employees are followed. In addition, procedures will provide for documentation of authorized deviations from those guidelines and that the documentation is available for NRC review.

TABLE 6.1-1

MINIMUM SHIFT CREW COMPOSITION

| POSITION | NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION | | |
|---|---|---|---|
| | ONE UNIT OPERATING | TWO UNITS OPERATING | TWO UNITS IN COLD SHUTDOWN OR REFUELING |
| S~~S~~ M | 1 | 1 | 1 |
| SRO | 1 | 1 | None |
| RO | 3 | 3 | 2 |
| AO | 4 | 4 | 4 |
| STA | 1 | 1 | None |

$\boxed{C9}$

Insert Table 6.1-1 (continued) information from page TS 6.1-5 move to Table 6.1-1

~~TABLE 6.1-1 (Continued)~~

S~~S~~ M - Shift ~~Supervisor~~ Manager with a Senior Reactor Operators License.

SRO - Individual with a Senior Reactor Operators License.

RO - Individual with a Reactors Operators License.

AO - Auxiliary Operator

STA - ~~Shift Technical Advisor~~ Individual providing advisory technical support to the unit operations shift crew

Except for the Shift ~~Supervisor~~ Manager the Shift Crew Composition may be one less than the minimum requirements of Table 6.1-1 for a period of time not to exceed 2 hours in order to accomodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.1-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift ~~Supervisor~~ Manager from the Control Room while the unit is in operation, an individual (other than the ~~Shift~~ technical advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift ~~Supervisor~~ Manager from the Control Room while the unit is shutdown or refueling, an individual with a valid SRO or RO license (other than the ~~Shift~~ technical advisor) shall be designated to assume the Control Room command functions.

C9 C11 C9 C9 C5 C12

Delete

C13

C. Organization units to provide a continuing review of the operational and safety aspects of the nuclear facility shall be constituted and have the authority and responsibilities outlined below:

1. Station Nuclear Safety and Operating Committee (SNSOC)

a. Function

The SNSOC shall function to advise the Site Vice President on all matters related to nuclear safety.

b. Composition

The SNSOC shall be composed of the:

| | |
|---|---|
| Chairman: | Manager - Station Safety and Licensing |
| Vice Chairman and Member: | Manager - Station Operations and Maintenance |
| Member: | Superintendent - Operations |
| Member: | Superintendent - Maintenance |
| Member: | Superintendent - Radiological Protection |
| Member: | Superintendent - Engineering |

c. Alternates

All alternate members shall be appointed in writing however, no more than two alternates shall participate as voting members in SNSOC activities at any one time.

d. Meeting Frequency

The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

e. Quorum

A quorum of the SNSOC shall consist of the Chairman or Vice Chairman and two members including alternates.

C13

f. Responsibilities

The SNSOC shall be responsible for:

1. Review of a) all new normal, abnormal, and emergency operating procedures and all new maintenance procedures, b) all procedure changes that require a safety evaluation, and c) any other procedures or changes thereto as determined by the Site Vice President which affect nuclear safety.

2. Review of all new test and experiment procedures that affect nuclear safety.

3. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

4. Review of proposed changes to Technical Specifications and shall submit recommended changes to the Site Vice President.

5. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and to the Management Safety Review Committee.

6. Review of all Reportable Events and special reports submitted to the NRC.

7. Review of facility operations to detect potential nuclear safety hazards.

8. Performance of special reviews, investigations or analyses and report thereon as requested by the Chairman of the SNSOC or Site Vice President.

Delete

C13

9. Deleted.

10. Deleted.

11. Review of every unplanned onsite release of radioactive material to the environs exceeding the limits of Specification 3.11, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Operations and to the Management Safety Review Committee.

12. Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.

13. Review of the Fire Protection Program and implementing procedures and shall submit recommended Program changes to the designated offsite management responsible for reviewing changes that pertain to Fire Protection.

g. Authority

The SNSOC shall:

1. Provide written approval or disapproval of items considered under (1) through (3) above. SNSOC approval shall be certified in writing by either the Manager - Station Operations and Maintenance or the Manager - Station Safety and Licensing.

2. Render determinations in writing with regard to whether or not each item considered under (1) through (5) above constitutes an unreviewed safety question.

3. Provide written notification within 24 hours to the Vice President - Nuclear Operations and to the Management Safety Review Committee of disagreement between SNSOC and the Site Vice President; however, the Site Vice President shall have responsibility for resolution of such disagreements pursuant to 6.1.A above.

h. Records

The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Operations and to the Management Safety Review Committee.

Delete

C13

2. MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

a. Function

The Management Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

1. Station Operations
2. Maintenance
3. Reactivity Management
4. Engineering
5. Chemistry and Radiochemistry
6. Radiological Safety
7. Quality Assurance Practices
8. Emergency Preparedness

b. Composition

The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANS 3.1 - (12/1979 Draft)

c. Alternates

All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

d. Consultants

Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

Delete

C13

e. Meeting Frequency

The MSRC shall meet at least once per calendar quarter.

f. Quorum

The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

g. Review

The MSRC shall be responsible for the review of:

1. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.

2. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.

3. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.

4. Proposed changes to Technical Specifications or the Operating Licenses.

Delete

C13

5. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.

6. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.

7. Events requiring written notification to the Commission.

8. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.

9. A representative sample of reports and meeting minutes of the SNSOC.

h. Audits

Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

1. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions

2. The performance, training and qualifications of the entire facility staff.

3. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.

Delete

C13

4. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.

5. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.

6. The Fire Protection Program and implementing procedures.

7. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.

8. The radiological environmental monitoring program.

9. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.

10. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive waste.

Delete

C13

i. Authority

The MSRC shall report to and advise the Senior Vise President - Nuclear on those areas of responsibility specified in Sections 6.1.c.2.g and 6.1.c.2.h.

j. Records

Records of MSRC activities shall be prepared, approved and distributed as indicated below:

1. Minutes of each MSRC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting.
2. Reports of reviews with safety significant findings encompassed by Section 6.1.c.2.g above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days following completion of the review.
3. Audit reports encompassed by Section 6.1.c.2.h above, shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

## 6.2 GENERAL NOTIFICATION AND REPORTING REQUIREMENTS

Specification

A. The following action~~s~~ shall be taken for Reportable Events:

1. A report shall be submitted pursuant to the requirements of Section 50.73 to 10 CFR~~, and~~.

delete ~~2. Each Reportable Event shall be reviewed by the SNSOC. The Vice President - Nuclear Operations and the MSRC shall be notified of the results of this review.~~ C13

B. Immediate notifications shall be made in accordance with Section 50.72 to 10 CFR.

C. CORE OPERATING LIMITS REPORT

Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle. Parameter limits for the following Technical Specifications are defined in the CORE OPERATING LIMITS REPORT:

1. TS 3.1.E and TS 5.3.A.6.b - Moderator Temperature Coefficient
2. TS 3.12.A.2 and TS 3.12.A.3 - Control Bank Insertion Limits
3. TS 3.12.B.1 and TS 3.12.B.2 - Power Distribution Limits

## 6.3 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

C1

Specification

A. The following actions shall be taken in the event a Safety Limit is violated:

1. The facility shall be placed in at least hot shutdown within 1 hour.

delete

2. The Safety Limit violation shall be reported to the Commission, the Vice President - Nuclear Operations, and the MSRC within 24 hours.

3. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

C14

4. The Safety Limit Violation Report shall be submitted to the Commission, the Vice President - Nuclear Operations, and the MSRC within 14 days of the violation.

2. The requirements of 6.4.B.1 above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr, but less than 500 rads/hr at one meter from a radiation source or any surface through which radiation penetrates. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift ~~Supervisor~~ manager on duty and/or the senior station individual assigned the responsibility for health physics and radiation protection. C

3. Written procedures shall be established, implemented, and maintained covering the activities referenced below:

   a. Process Control Program implementation.

   b. Offsite Dose Calculation Manual implementation.

delete

~~C. All procedures described in 6.4.A and 6.4.B shall be reviewed and approved by the Station Nuclear Safety and Operating Committee (SNSOC) prior to implementation. Subsequent procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC prior to implementation.~~ C13 ~~All other changes shall be independently reviewed and approved as discussed in the Updated Final Safety Analysis Report.~~ C15

D. All procedures described in Specifications 6.4.A and 6.4.B shall be followed.

E. The facility Fire Protection Program and implementing procedures which have been established for the station shall be implemented and maintained.

F. Deleted

Delete

~~G. In cases of emergency, operations personnel shall be authorized to depart from approved procedures where necessary to prevent injury to personnel or damage to the facility. Such changes shall be documented, reviewed and approved by the Station Nuclear Safety and Operating Committee.~~ C16 C13

G. Deleted

## 6.8 PROCESS CONTROL PROGRAM AND OFFSITE DOSE CALCULATION MANUAL

A. Process Control Program (PCP)

Changes to the PCP:

1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.5.B.12. This documentation shall contain:

    a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

    b. A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

2. Shall require ~~review and acceptance by the SNSOC and~~ the approval of the ~~Site Vice President~~ plant manager prior to implementation. C13 C5

B. Offsite Dose Calculation Manual (ODCM)

Changes to the ODCM:

1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.5.B.12. This documentation shall contain:

    a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

b. A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

2. Shall require ~~review and acceptance by the SNSOC and~~ the approval of the ~~Site Vice President~~ plant manager prior to implementation. C13 C5

3. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

**Attachment 3**

**Proposed Technical Specifications**

**Dominion**
**Surry Power Station Units 1 and 2**

4. The reactor thermal power level shall not exceed 118% of rated power.

B. In the event the Safety Limit is violated, the facility shall be placed in at least HOT SHUTDOWN within 1 hour. The safety limit is exceeded if the combination of Reactor Coolant System average temperature and thermal power level is at any time above the appropriate pressure line in TS Figures 2.1-1, 2.1-2 or 2.1-3; or the core thermal power exceeds 118% of the rated power.

Basis

To maintain the integrity of the fuel cladding and prevent fission product release, it is necessary to prevent overheating of the cladding under all operating conditions. This is accomplished by operating within the nucleate boiling regime of heat transfer, wherein the heat transfer coefficient is very large and the clad surface temperature is only a few degrees Fahrenheit above the reactor coolant saturation temperature. The upper boundary of the nucleate boiling regime is termed Departure From Nucleate Boiling (DNB) and at this point there is a sharp reduction of the heat transfer coefficient, which would result in high clad temperatures and the possibility of clad failure. DNB is not, however, an observable parameter during reactor operation. Therefore, DNB has been correlated to thermal power, reactor coolant temperature and reactor coolant pressure which are observable parameters. This correlation has been developed to predict the DNB flux and the location of DNB for axially

## 2.2 SAFETY LIMIT, REACTOR COOLANT SYSTEM PRESSURE

Applicability

Applies to the maximum limit on Reactor Coolant System pressure.

Objective

To maintain the integrity of the Reactor Coolant System.

Specification

A. The Reactor Coolant System pressure shall not exceed 2735 psig with fuel assemblies installed in the reactor vessel.

B. In the event the Safety Limit is violated, the facility shall be placed in at least HOT SHUTDOWN within 1 hour.

Basis

The Reactor Coolant System(1) serves as a barrier which prevents radionuclides contained in the reactor coolant from reaching the environment. In the event of a fuel cladding failure the Reactor Coolant System is the primary barrier against the release of fission products. The maximum transient pressure allowable in the Reactor Coolant System pressure vessel under the ASME Code, Section III is 110% of design pressure. The maximum transient pressure allowable in the Reactor Coolant System piping, valves and fittings under USAS Section B31.1 is 120% of design pressure. Thus, the safety limit of 2735 psig (110% of design pressure) has been established.(2)

# 6.0 ADMINISTRATIVE CONTROLS

## 6.1 Responsibility, Organization and Qualifications

### 6.1.1 Responsibility

1. The plant manager shall be responsible for the overall unit operation. During his absence, the plant manager will delegate in writing the succession to this responsibility.

2. The Shift Manager shall be responsible for the control room command function.

### 6.1.2 Organization

1. Onsite and Offsite Organizations

   Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

   a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the QA Program. The plant-specific titles of those personnel fulfilling the responsibilities of the positions delineated in these Technical Specifications shall be maintained in appropriate administrative documents.

   b. The plant manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

   c. A specified corporate officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining and providing technical support to the plant to ensure nuclear safety.

   d. The individuals who train the operating staff, carry out health physics, or perform quality assurance functions may report to the appropriate onsite manager; however, these individuals shall have sufficient organizational freedom to ensure independence from operating pressures.

2. Unit Staff

   The unit staff organization shall include the following:

   a. Each on-duty shift shall be composed of at least the minimum shift crew composition for each unit as shown in Table 6.1-1.

   b. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the position.

   c. All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator limited to fuel handling who has no other concurrent responsibilities during this operation.

   d. The operations manager shall hold (or have previously held) a Senior Reactor Operator License for Surry Power Station or a similar design Pressurized Water Reactor plant. The Supervisor Nuclear Shift Operations shall hold an active Senior Reactor Operator License for Surry Power Station.

   e. Procedures will be established to insure that NRC policy statement guidelines regarding working hours established for employees are followed. In addition, procedures will provide for documentation of authorized deviations from those guidelines and that the documentation is available for NRC review.

6.1.3 Unit Staff Qualifications

1. Each member of the unit staff shall meet or exceed the minimum qualifications as specified in the Quality Assurance Program. Incumbents in the positions of Shift Manager, Unit Supervisor (SRO), Control Room Operator (RO), and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4).

2. For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator and a licensed Reactor Operator are those individuals who, in addition to meeting the requirements of TS 6.1.3.1 perform the functions described in 10 CFR 50.54(m).

TABLE 6.1-1

MINIMUM SHIFT CREW COMPOSITION

| POSITION | NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION | | |
|---|---|---|---|
| | ONE UNIT OPERATING | TWO UNITS OPERATING | TWO UNITS IN COLD SHUTDOWN OR REFUELING |
| SM | 1 | 1 | 1 |
| SRO | 1 | 1 | None |
| RO | 3 | 3 | 2 |
| AO | 4 | 4 | 4 |
| STA | 1 | 1 | None |

SM - Shift Manager with a Senior Reactor Operators License.

SRO - Individual with a Senior Reactor Operators License.

RO - Individual with a Reactors Operators License.

AO - Auxiliary Operator

STA - Individual providing advisory technical support to the unit operations shift crew.

Except for the Shift Manager, the Shift Crew Composition may be one less than the minimum requirements of Table 6.1-1 for a period of time not to exceed 2 hours in order to accomodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.1-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Manager from the Control Room while the unit is in operation, an individual (other than the technical advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Manager from the Control Room while the unit is shutdown or refueling, an individual with a valid SRO or RO license (other than the technical advisor) shall be designated to assume the Control Room command functions.

6.2 GENERAL NOTIFICATION AND REPORTING REQUIREMENTS

Specification

A. The following action shall be taken for Reportable Events:

A report shall be submitted pursuant to the requirements of Section 50.73 to 10 CFR.

B. Immediate notifications shall be made in accordance with Section 50.72 to 10 CFR.

C. CORE OPERATING LIMITS REPORT

Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle. Parameter limits for the following Technical Specifications are defined in the CORE OPERATING LIMITS REPORT:

1. TS 3.1.E and TS 5.3.A.6.b - Moderator Temperature Coefficient
2. TS 3.12.A.2 and TS 3.12.A.3 - Control Bank Insertion Limits
3. TS 3.12.B.1 and TS 3.12.B.2 - Power Distribution Limits

Section 6.3, "Action to Be Taken if a Safety Limit Is Exceeded," has been relocated, in part, to Section 2.1 and Section 2.2. Specific reporting requirements have been removed from TS.

2. The requirements of 6.4.B.1 above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr, but less than 500 rads/hr at one meter from a radiation source or any surface through which radiation penetrates. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Manager on duty and/or the senior station individual assigned the responsibility for health physics and radiation protection.

3. Written procedures shall be established, implemented, and maintained covering the activities referenced below:

    a. Process Control Program implementation.

    b. Offsite Dose Calculation Manual implementation.

D. All procedures described in Specifications 6.4.A and 6.4.B shall be followed.

E. The facility Fire Protection Program and implementing procedures which have been established for the station shall be implemented and maintained.

F. Deleted

G. Deleted

## 6.8 PROCESS CONTROL PROGRAM AND OFFSITE DOSE CALCULATION MANUAL

A. Process Control Program (PCP)

Changes to the PCP:

1. Shall be documented and records of reviews performed shall be retained as required by the Operational Quality Assurance Program Topical Report. This documentation shall contain:

    a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

    b. A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

2. Shall require the approval of the plant manager prior to implementation.

B. Offsite Dose Calculation Manual (ODCM)

Changes to the ODCM:

1. Shall be documented and records of reviews performed shall be retained as required by the Operational Quality Assurance Program Topical Report. This documentation shall contain:

    a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

b. A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

2. Shall require the approval of the plant manager prior to implementation.

3. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.