

10 CFR 50.55a

2130-04-20214
September 8, 2004

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Oyster Creek Generating Station
Facility License No. DPR-16
Docket No. 50-219

Subject: Response to Request for Additional Information Concerning
Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel

- References:
- 1) AmerGen letter 2130-00-20300 dated November 10, 2000, "Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel"
 - 2) AmerGen letter 2130-00-20304 dated November 14, 2000, "Modification to Proposed Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel"
 - 3) USNRC letter dated November 16, 2000, "Request to Use an Alternative Repair of the Control Rod Drive Housing Interface with the Reactor Vessel at the Oyster Creek Nuclear Generating Station (TAC NO. MB0461)"
 - 4) AmerGen letter 2130-01-20031 dated January 19, 2001, "Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel Clarification of Leakage Inspection"
 - 5) USNRC letter dated January 8, 2002, "Oyster Creek Nuclear Generating Station – Clarification of Leakage Inspection (TAC NO. MB1065)"
 - 6) AmerGen letter 2130-02-20214 dated July 26, 2002, "Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel"
 - 7) AmerGen letter 2130-02-20291 dated October 4, 2002, "Additional Information - Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel (TAC No. MB5700)"
 - 8) USNRC letter dated October 18, 2002, "Oyster Creek Nuclear Generating Station - Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel (TAC NO. MB5700)"
 - 9) AmerGen letter 2130-03-20271 dated October 21, 2003, "Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel"
 - 10) AmerGen letter 2130-04-20157 dated July 20, 2004, "Response to Request for Additional Information Concerning Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel"
 - 11) AmerGen letter 2130-04-20201 dated August 23, 2004, "Response to Request for Additional Information Concerning Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel"

A047

U.S. Nuclear Regulatory Commission
September 8, 2004
Page 2

In the Reference 9 letter, in accordance with 10 CFR 50.55a(a)(3)(i), AmerGen Energy Company, LLC (AmerGen) requested continued approval of the proposed alternative to 10 CFR 50.55a(g) as contained in the Reference 6 letter above.

This issue was discussed with the NRC staff in conference calls dated May 26, 2004, August 6, 2004, August 17, 2004, and August 19, 2004, and resulted in additional information being provided in the Reference 10 and 11 letters. Attachment 1 contains our response to additional questions discussed during these calls.

If you should have any questions, please contact Mr. Tom Loomis at 610-765-5510.

Very truly yours,



M. P. Gallagher
Director – Licensing & Regulatory Affairs
AmerGen Energy Company, LLC
Exelon Generation Company, LLC

Attachment: Response to Request for Additional Information

cc: S. J. Collins, USNRC, Administrator, Region I
P. S. Tam, USNRC, Senior Project Manager, Oyster Creek
R. J. Summers, USNRC, Senior Resident Inspector, Oyster Creek
File No. 00086

ATTACHMENT 1

Response to Request for Additional Information Concerning Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel

Question:

1. If additional roll repairs of housing penetrations and/or re-roll of the housing penetrations 42-43 and 46-39 are performed during the R20 outage, will you perform VT-1 of the stub tube and stub tube-to-vessel attachment weld of these additional rolled and/or re-rolled repaired penetrations at the next refueling outage (i.e., R21)?

Response:

Yes.

Question:

2. What leakage rates, if any, will be used for the repaired housing penetrations during R20?

Response:

Refer to the Tables provided in the Reference 11 letter (AmerGen letter 2130-04-20201 dated August 23, 2004, "Response to Request for Additional Information Concerning Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel").

Question:

3. Confirm that after cycle R20, if the ASME Committee finds the proposed code case unacceptable, an ASME Code repair (weld repair) of the roll repaired housings will be performed at the next schedule refueling outage.

Response:

Refer to the Tables provided in the Reference 11 letter (AmerGen letter 2130-04-20201 dated August 23, 2004, "Response to Request for Additional Information Concerning Alternative Repair of Control Rod Drive Housing Interface with Reactor Vessel").

Question:

4. Confirm that Oyster Creek will continue the HWC program through the R21 outage to support the extension of these repairs.

Response:

Oyster Creek Generating Station will continue to comply with the Hydrogen Water Chemistry Program through the R21 outage.