

September 16, 2004

LICENSEE: Southern Nuclear Operating Company

FACILITY: Joseph M. Farley Nuclear Plant, Units 1 and 2

SUBJECT: SUMMARY OF A TELEPHONE CONFERENCE HELD ON
SEPTEMBER 14, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY
COMMISSION AND THE SOUTHERN NUCLEAR OPERATING COMPANY
CONCERNING THE REVIEW FOR THE JOSEPH M. FARLEY NUCLEAR
PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION (TAC NOS.
MC0774 AND MC0775)

The U.S. Nuclear Regulatory Commission staff and representatives of Southern Nuclear Operating Company (SNC or the applicant) held a telephone conference on September 14, 2004, to discuss questions pertaining to the Joseph M. Farley Nuclear Plant (FNP) license renewal application.

The conference call was useful in clarifying the intent of the staff's questions. On the basis of the discussion, the applicant was able to better understand the staff's questions. No staff decisions were made during the telephone conference, and the applicant agreed to provide information for clarification.

Enclosure 1 provides a list of the telephone conference participants. Enclosure 2 contains a listing of the questions discussed with the applicant, including a brief description on the status of the item. The applicant has had an opportunity comment on this summary.

/RA/

Tilda Y. Liu, Senior Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-348 and 50-364

Enclosures: As stated

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September 14, 2004

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Jan Fridrichsen
Mike Macfarlane
Lou Bohn
Charles Pierce

Affiliation

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Southern Nuclear Operating Company (SNC)
SNC
SNC
SNC

REVIEW OF LICENSE RENEWAL APPLICATION (LRA) FOR FARLEY UNITS 1 AND 2

September 14, 2004

On September 13, 2004, the staff sent the following questions via D-RAI 3.2-8 to the applicant. The staff's questions and the associated discussion with the applicant held on September 14, 2004 are presented below:

Supplemental Information on the Aging Management Review Results for the High Head (Charging Pump) Casings [from applicant's letter dated August 31, 2004]

D-RAI 3.2-8

FNP's six charging/high head safety injection pumps are susceptible to loss of material due to clad cracking and subsequent corrosion in the carbon steel base metal. The applicant stated that the pump casing inspections are existing periodic tasks that manage loss of material in the carbon steel base metal resulting from boric acid corrosion at locations of clad cracking. Inspections include periodic non-intrusive ultrasonic testing (UT) exams of accessible areas of the pump casings to detect through-clad cavities (loss of material) in the carbon steel base material. Visual testing (VT) examination of the internal surface of the pump casings is performed any time a rotating assembly is removed during pump maintenance. To further evaluate the aging management program for the pumps, the staff requests the applicant to provide the following information:

- a. The applicant stated that pump 2A casing received a VT-1 for clad cracking during inspection of the rotating assembly; however, it is not clear if the other five pumps received VT-1 inspections. Explain if VT-1 inspections for clad cracking have been performed (or are scheduled to be performed) for the 1A, 1B, 1C, 2B, and 2C pumps. Are all internal clad surfaces of the pump casing VT-1 inspected? Will VT-1 inspections be performed during the period of extended operation?
- b. The applicant stated that UT inspections will be performed to detect loss of material in the carbon steel base material of the pump casings. Since the loss of material is not limited to a specific location, describe the process used to ensure that the UT inspection methodology (including sample size) will detect the loss of material in the pump casings.
- c. NUREG-1800, Section A.1.2.3.1 recommends that the scope of the aging management program include the specific structures and components of which the program manages the aging. The applicant is requested to include the charging/high head safety injection pumps in the scope of the program for Periodic Surveillance and Preventive Maintenance Activities.
- d. NUREG-1800, Section A.1.2.3.5 recommends that monitoring and trending activities be described, and they should provide predictability of the extent of degradation and thus affect timely corrective actions. Plant-specific and/or industry operating experience may be considered in evaluating the appropriateness of the technique and frequency. The applicant is requested to describe the monitoring and trending process for the Periodic Surveillance and Preventive Maintenance Activities.

Enclosure 2

e. NUREG-1800, Section A.1.2.3.6 recommends that acceptance criteria and its basis be described. The applicant is requested to describe the UT acceptance criteria and its basis for the Periodic Surveillance and Preventive Maintenance Activities.

f. NUREG-1800, Section A.1.2.3.10 recommends that the operating experience provide objective evidence to support the conclusion that the effects of aging will be managed adequately so that the intended function will be maintained during the period of extended operation. The applicant is requested to provide plant-specific and/or industry operating experience for corrosion of the pump casings to support the conclusion that the aging effects will be adequately managed by the Periodic Surveillance and Preventive Maintenance Activities.

Discussion: The applicant indicated that the questions are clear. This D-RAI will be sent as a formal RAI.

Joseph M. Farley Nuclear Plant

cc:

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