

September 3, 2004

Mr. David A. Christian  
Sr. Vice President and Chief Nuclear Officer  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060-6711

SUBJECT: SURRY POWER STATION, UNITS 1 AND 2 - REQUEST FOR ADDITIONAL  
INFORMATION (RAI) REGARDING ANNUAL STEAM GENERATOR  
INSERVICE INSPECTION SUMMARY REPORT

Dear Mr. Christian:

By letter dated February 23, 2004, Virginia Electric and Power Company (VEPCO) submitted the annual steam generator inservice inspection summary report for Surry Power Station, Units 1 and 2. The NRC staff has completed its review of VEPCO's submittal and has determined that additional information is required in order to complete its evaluation.

Our questions are provided in the Enclosure. The NRC staff requests a response to the RAI within 60 days of the date of this letter.

Sincerely,

*/RA/*

Stephen R. Monarque, Project Manager, Section 1  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosure: As stated

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REQUEST FOR ADDITIONAL INFORMATION

ANNUAL STEAM GENERATOR INSERVICE INSPECTION SUMMARY REPORT

SURRY POWER STATION, UNITS 1 & 2

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NUMBERS 50-280 & 50-281

1. The Plugging/Repair Record table on page 3 of Attachment 1 to your submittal dated February 23, 2004, indicates that 7 tubes are plugged in the B steam generator of Unit 1. A historical review of the 2000 and 2001 inservice inspection summary reports indicates that 14 tubes were plugged in the B steam generator prior to the 2003 inspection. Please verify the total number of tubes plugged in Unit 1, Steam Generator B following the 2003 steam generator tube inspections.
2. The February 23, 2004, submittal indicates that the steam generators at both units have experienced denting in peripheral tubes near the 6<sup>th</sup> and 7<sup>th</sup> tube support plates. Please discuss whether new dents were identified during your 2003 inspections and whether existing dents have "grown." If existing dents are increasing in magnitude, please discuss how this condition (i.e., increased stresses and associated increase in the propensity for stress corrosion cracking) was addressed in your operational assessment.
3. The indications identified at the top of tubesheet for hot leg locations R22C82 and R23C82 were attributed to interaction with a foreign object that is believed to no longer be in the area. These indications were identified during an inspection of the "critical area" of the hot leg top of tubesheet. Please discuss the nature of this critical area (i.e., the low velocity region in the middle of the tube bundle). It was assumed that the object was no longer in this area. Please discuss the basis for this conclusion. Specifically address whether a visual inspection was performed in the critical area and the possibility that the part was not in contact with the tube during the eddy current inspections (or does not conduct eddy currents). Please discuss how other forms of degradation, such as intergranular attack and closely spaced pits, were ruled out as possible causes of the indications.
4. A search of our document management system indicates that the NRC staff does not have a "15-day" plugging report on file for the 2003 Unit 2 steam generator tube inspection. Please provide another copy of this report.

Enclosure

Surry Power Station, Units 1 & 2

cc:

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