# **Final Submittal**

# BROWNS FERRY EXAM 50-259, 50-260, & 50-296/2004-301

April 23 - 30, 2004

 As Given Simulator Scenario Operator Actions ES-D-2

# 

# Scenaros

(Browns Ferry 2004-301)

Facility: BFN	Scenario Number:	HLT0212#2	Op-Test Number:
	Final		
Examiners: Tim Kolb Ron Aiello	)	Operators:	

Initial Conditions: IC 28, 100% pwr

Turnover: Reduce power to 80% and remove 2B RFP from service for maintenance. 2A CRD pump in standby awaiting PMT after speed increaser inspection, place pump in service when MM is ready for PMT.

Event	Malfunction	Event	Event
Number	Number	Type*	Description
1	None	R-RO	Reduce power with Recirc to allow
		R-SRO	
		N-RO	removal of 2B Rx Feed pump from service.
	·	N-SRO	
2	Ed09a	C-RO	Normal feeder breaker trip and Bd lockout for 4kv
		C-BOP	Shutdown Bd A. ITS for SRO, Transfer A 480v
		C-SRO	Diesel Aux board for BOP. RO responds to loss of
			CRD pump from power loss.
3	Rd08	C-SRO	Accumulator trouble for an HCU. SRO must direct
			AUO to recharge Accumulator and refer to ITS for
	·		required actions.
4	Th10a 100	C-RO	Sequential failure of Recirc pump A seals. Time
	1:00	C-SRO	delay allows proper response for 1 Seal failure with
	Th11a 100		subsequent failure of second seal. Seal failures will
	15:00		require isolation of Recirc pump. SRO makes SLO
			determination in ITS.
5	Tc07	M-all	EHC leak with Turbing Trip on loss of Hadamilla
J	Bat atws95	ivi-aii	EHC leak with Turbine Trip on loss of Hydraulic pressure with a 95% ATWS on east side rods, EOI-1
	east		T
	Casi		and C-5 entry, forces lower level for power control.
6	Th23 20	C-All	Fuel failure, resultant Rad levels will cause entry into
Ü	1112020	O 7111	EOI-3

<sup>\* (</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Appendix .	D2	
Op Test _	2004NRC	Scenario2 Event No1 Page _1_ of _1
Event Des	scription: Redu	ce Reactor Power to 80% and Remove the 2B RFP from
Time	Position	Applicant's Action or Behavior
	SRO	Notifies ODS of Power Reduction
	SRO	Notifies Chemistry and radcon of power reduction
	BUO	Begins power drop with recirculation flow per GOI-100-12
		<ul> <li>Decreases Recirc flow per OI-68 section 6.2</li> </ul>
		<ul> <li>Utilizing slow ( medium ) decrease PB</li> </ul>
		• Stops power reduction @ 80% power.
	DUO	Performs peer check of power reduction
	BUO	Removes the 2B RFP from service per OI-3

Appendix		
Op Test_	2004NRC	Scenario2
Event De	scription: Loss	of 4 KV Shutdown Board " A " ( Bus Lockout )
Time	Position	Applicant's Action or Behavior
	Crew	Identifies/Reports a loss of the 4 KV Shutdown Board "A"
	DUO	Recognizes "A "D/G starts and fails to load
	BUO	Recognizes 1B CRD Pump Trip
	SRO	Directs placing 2A CRD Pump Inservice per AOI 85-3
	DUO	Dispatches AUO to 4 KV Shutdown Board "A "
	SRO	Directs "A" D/G Shutdown and AUO to monitor while running.  • Directs Diesel Aux Board Transfer per OI-57B step 8.10  • Contacts Electrical Maintenance to troubleshoot the 4 KV Shutdown Board
	BUO	Places 2A CRD Pump in service per AOI-85-3
	DUO	Identifies loss of power to RHR 2A, CS 2A, and RHR SW A1
	SRO	Enters T.S. LCO 3.8.7 and verify no loss of safety function exists per ODP-8.
	DUO	Transfers Diesel Aux Board "A per OI-57B"  • Verifies "A" & "C" SGT Starts and Refuel ventilation Isolates  • Shuts Down "A" D/G per OI-82

Appendix I	D2	
Op Test _	2004NRC	Scenario2 Event No3 Page _1 _ of _1 _
Event Des	scription: Cont	rol Rod HCU Low Gas Pressure
Time	Position	Applicant's Action or Behavior
	BUO	Reports alarm "CRD Accum Press Low/Level High ", identifies HCU as 30-55  • Refers to ARP. dispatches AUO to CRD Module
,	SRO	Refers to T.S. 3.1.5, Declares Control Rod 30-55 slow within 8 hours  • Determines no further actions required by T.S.
	BUO	Directs AUO to recharge accumulator per OI-85
	SRO	Exits LCO 3.1.5 when HCU is returned to normal

Appendix D2	?					
Op Test	_2004NRC	Scenario	2	Event No4	l Page	e _1_ of_2

## Event Description: Failure of both seals on the 2A Recirc Pump

Time	Position	Applicant's Action or Behavior
	BUO/DUO	Recognizes and responds to the 2A Recirc Pump inboard seal failure per ARP 2-FA-68-62 and reports it to the SRO
	SRO	Directs DUO to monitor 2A Recirc Pump Seal Pressures per ARP and record data per ARP 2-ARP-9-4A
	DUO	Recognizes increase in Drywell Pressure and Temperature and reports it to the SRO.
	SRO	SRO directs DUO to vent the Drywell per AOI-64-1
	DUO	Places additional drywell cooling in service and vents Drywell per 2-AOI-64-1 step 4.2.2.3
	BUO	Recognizes failure of the outboard seal on the 2A Recirc pump and reports it to the SRO (Both seals fail).
	SRO	Directs 2A Recirc Pump tripped and isolated. per 2-ARP -9-4A
	BUO	Trips and isolates the 2A Recirc Pump per 2 -ARP-9-4A
	SRO	<ul> <li>Directs the following actions per AOI-68-1B,</li> <li>Monitor Power/Flow Map to determine region in the Power to Flow Map</li> <li>Maintain operating Recirc Pump less than 46,600 gpm</li> <li>Notifies Reactor Engineer</li> <li>Ensures Recirc Pump suction and discharge valve closed</li> </ul>

Appendix D2	?	
Op Test	_2004NRC	Scenario2 Event No4 Page _2 _ of _2
Event Descr	ription: Failt	ure of both seals on the 2A Recirc Pump (Continued)
Time	Position	Applicant's Action or Behavior
	SRO	Control Rods Inserted to comply with the Power Directs /Flow Map
	BUO	Ensures /Maintains operating Recirc Pump Speed less than 46,600 gpm
	BUO	Inserts Control Rods per the Emergency Shove Sheet to exit region 2 of the Power/Flow Map
	SRO	Directs Reactor Engineer to perform surveillances/actions necessary to support Single Loop Operation
		<ul> <li>2-SR-3.4.1 (Single Loop Operation)</li> <li>2-TI-248 (Core Flow Determination in Single Loop Operation</li> </ul>
	SRO	Notifies ODS

Appendix	D2				
Op Test_	NRC2004	Scenario2 Event No5 Page _1_ of _1_			
Event De	Event Description: EHC Leak/ SCRAM				
Time	Position	Applicant's Action or Behavior			
	DUO	Recognizes and reports "EHC Reservoir Low Level Alarm			
		Responds per ARP 2-XA-55-7B			
		<ul> <li>Monitors EHC pump amps and discharge pressure</li> </ul>			
		<ul> <li>Dispatches AUOs to locate EHC leakage and add</li> </ul>			
		EHC Hydraulic fluid if necessary			
	SRO	Assign dedicated operator to monitor the EHC system per the ARP			
	SRO	Contacts Environmental Group to help contain EHC Fluid			
		<ul> <li>Takes action to ensure isolation of Floor Drains</li> </ul>			
		associated with any EHC Leakage			
	DUO	Reports Low EHC Pressure			
	SRO	Directs Manual Scram of the Reactor			
		Directs Actions per AOI 100-1			
		•			
	BUO	Inserts Manual Scram per AOI-100-1			

Appendix	D2	
		Scenario2 Event No6 Page _1_ of _3
	scription: ATWS	
Time	Position	Applicant's Action or Behavior
	BUO	Reports all rods not in and performs the following:  Mode Switch to Shutdown  Initiates ARI  Recognizes and reports power above 5% and Recirc pumps not tripped  Reports MSIVs are open  Inserts SRMs/IRMS
	SRO	Enters EOI-1 and C5 and directs/ensures the following:  • Mode Switch to Shutdown  • ARI initiated  • Recirc Pumps tripped
	SRO	Directs ADS Inhibited
	DUO	Inhibits ADS
	SRO	Declares Site Area Emergency, EAL 1.2-S
	SRO	Directs implementation of Appendix 4 to lower Reactor Water Level to less than -50"
	BUO	Maintains Reactor Water Level -50" to -100" with RFPs, RCIC, HPCI
	DUO	Performs EOI Appendix 4, Locks out HPCI
	SRO	Directs Appendix 1F, 2, and 1D be performed
	SRO	Directs Appendices 8A, 8C, 8E & 8G be performed
	BUO/DUO	Recognizes turbine trip and loss of Bypass Valves as accumulators depressurize

Appendix		
Op Test_	2004NRC	Scenario2 Event No6 Page 2_ of _3_
Event De	scription: ATW	/S/Fuel failure
Time	Position	Applicant's Action or Behavior
	SRO	Directs Reactor Pressure maintained less than 1073 psig
		with one or more of the following:
		• MSRVs
		Main Steam Line Drains
		• RFPs
	DUO	Recognizes and reports OG Pretreatment Rad High Alarm
	DUO	Maintains Reactor Pressure less than 1073 with one or more
		of the following:
		• MSRVs
		<ul> <li>Main Steam Line Drains (Appendix 11D)</li> </ul>
		• RFP ( App 11F )
	BUO	Directs AUO to close 2-85-586
	DUO	Recognizes and reports the Reactor Area Radiation
		Monitors are in alarm and are increasing.
	SRO	Orders and Call David D 111
	SKO	Orders evacuation of the Reactor Building
		Notifies HP of increasing Rad levels
		• Ensures that announcements are made to notify plant
		personnel.
		Directs DUO to continue to monitor Plant radiation levels
		icveis
	SRO	Directs Suppression Pool Cooling be placed in service per
		appendix 17A
	DUO	Places Suppression Pool Cooling in service per appendix
		17A
	SRO	Directs SLC Injection BEFORE 110 deg Torus Temp.
	BUO	When field operators report that the 2-85-586 is closed:
	-	Bypasses RWM
	ĺ	<ul> <li>Drives Rods utilizing EMERGENCY IN in a spiral</li> </ul>
		pattern per Appendix 1D

Time Position Applicant's Action or Behavior  SRO Directs BUO to complete the actions required in Appendix 1F  BUO Resets Reactor Scram  • Verifies SDV Vents and Drains open • Monitors SDV High Level Alarms  BUO Recognizes that SDV is drained: • SDV High Level Alarms Clear • Reports to SRO  BUO Directs AUOs to re-open the 2-85-586 valve per Appendix 1F  SRO Directs BUO to insert a manual scram per Appendix 1F  BUO Initiates Reactor Scram after the 2-85-586 valve has been re-opened • Verifies HCU Accumulators have been recharged • Depresses Scram PB • Verifies Control Rod Motion • Reports All Rods In	Appendix	D2	
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Reports All Rods In			Depresses Scram PB
			Verifies Control Rod Motion
SRO Exits EOI-1, RC/O, Directs SLC Injection terminated if	-		Reports All Rods In
		SRO	Exits EOI-1, RC/Q, Directs SLC Injection terminated if
SLC was initiated			SLC was initiated
SRO Directs OSUS to perform Appendix 16F & 16G			Directs OSUS to perform Appendix 16F & 16G
SRO Directs RPV Water level maintained between -162" and		SRO	Directs RPV Water level maintained between -162" and
level at which termination and prevention stopped.			level at which termination and prevention stopped.
BUO Maintains RPV level within band as directed by SRO		BUO	Maintains RPV level within band as directed by SRO

	vity Control Plan		Issued			Expires
Unit 2 Power Reduction		Today			Next	Month
	Name		Office Ext.	Home Pho	ne	Pager
On-Call	IM ENG			1	T	7 4501
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	Sequence Exchan				<u> </u>	
	Control Rod Patte	ern Adjustm	ent (describe in	purpose	<u> </u> :	
	Surveillance Testi	ing (describ	e in purpose)			
<del>- A</del> -	Maintenance/Mod		escribe in purpo	ese)	IMS	
	Other (describe in	i purpose)	****			
		·			***************************************	
Assumptions	/Hold Points:					
/ * / / /	r to 80% with flow pe	er OI-68			-	
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POWER / FLOW MAP.		
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IMRE	Date	today
YOUMRE	Date	today
	Date	
	Date	
	Date	
	e POWER / FLOW MAP.	IMRE Date

Facility: Bl	FN	Scenario Number:	HLT0212#3	Op-Test Number:
			Final	
Examiners:	Tim Kolb Ron Aiello		Operators	3:
•	_ICOII / EICHO			***************************************
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Initial Conditions: IC 27, 90% power

Turnover: 2-FCV-23-34 tagged for repairs. 7 day LCO's 3.7.1, 3.6.2.3, 4, 5 entered 6 hours earlier. Repairs forecast to be complete in 12 hours. 1B CRD pump tagged for inspection. Rx power was lowered to 90% last shift to allow Turbine valve testing that has been completed. Return power to 100% at 10 MWe/min with recirc flow.

	,	,	
Event	Malfunction	Event	Event
Number	Number	Type*	Description
1	None	R-RO R-SRO	Restore power to rated with Recirc Flow.
2	Th30v 0	C-SRO	Fails 2-LIS-3-203B downscale with expected half scram. RO responds to half scram, SRO refers to ITS
3	Fw30a	C-RO	A Feed pump flow controller fails low requiring removal of A RFW pump and power reduction.
4	Rd25	C-RO C-SRO	Rod 14-35 position indication fails requiring inserting rod to 46. RO must insert Rod. SRO refers to ITS.
5	Th23 35 15:00	C-RO C-SRO	35% fuel failure ramped in over 15 minutes. RO reduces power when instructed due to rising Rad levels
6	Th35b 8 8:00 Th23 80 1:00 Rp15a Rp15b Rc08	M-all	Steam leak from B main steam line outside containment. Fuel failure increased to 80%. SRO enters EOI-3 due to high Rad/Temp and then EOI-1 when an area reaches max safe. RO inserts manual scram. MSIV's fail to close on High Temp. RCIC steam supply valve failure to open on initiation signal.
	Ad01m 100	С	SRV sticks open, take actions per AOI
	Hp07 Rd01a	С	HPCI inverter power failure and trip of CRD pump. This removes available high pressure makeup and requires crew to implement the actions of Contingency C-1.

<sup>\* (</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

#### SIMULATOR EVALUATION GUIDE

TITLE :		RAISE POWER WITH RECIRC, RPS LOW LEVEL IN FAILURE, RFP FAILURES, RPIS FAILURE FOR A ROFAILURE, MAIN STEAM LEAK, MSRV FAILS OPEN, FAILURE, RCIC TRIP, CRD PUMP FAILURE, CONTIL EMERGENCY DEPRESSURIZE BEFORE LEVEL RE	OD, FUEL HPCI INVERTER NGENCY C1,	<u>}</u>
REVISION	:	0		
DATE	:	April 1, 2004		
PROGRAM	:	BFN Operator Training		
PREPARED	BY:_	(Operations Instructor)	\ Date	
		(Operations instructor)	Date	
REVIEWED	BY:	(Lead Examiner or Designee)	1	
		(Lead Examiner or Designee)	Date	
REVIEWED	BY:		\	
		(Operations Training Manager or Designee)	Date	
CONCURRE	:D:		1	
		(Operations Superintendent or Designee)	Date	****
VALIDATION BY:	N		١	
		(Operations US) (Required for Exam Scenarios only)	Date	

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	NUCLEAR TRAINING					
	REVISION/USAGE LOG					
REVISION		DATE	PAGES	REVIEWED/REVISED		
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I. Program: BFN Operator Training

II. Course: Examination Guide

III. Title: RAISE POWER WITH RECIRC, RPS LOW LEVEL INSTRUMENT

FAILURE, RFP FAILURES, RPIS FAILURE FOR A ROD, FUEL

FAILURE, MAIN STEAM LEAK, MSRV FAILS OPEN, HPCI INVERTER FAILURE, RCIC TRIP, CRD PUMP FAILURE, CONTINGENCY C1, EMERGENCY DEPRESSURIZE BEFORE LEVEL REACHES -185"

IV. Length of Scenario: 11/2 hours

V. Examination Objectives:

A. Terminal Objective

- 1. Perform routine shift turnover, plant assessment and routine shift operation in accordance with BFN procedures.
- 2. Given abnormal conditions, the operating crew will place the unit in a stabilized condition per normal, abnormal, annunciator and emergency procedures.

#### B. Enabling Objectives:

- 1. The operating crew will raise power with recirc IAW GOI-100-12.
- 2. The operating crew will recognize and respond to an inadvertent half scram due to instrument failure in accordance with ARPs and Technical Specifications.
- 3. The operating crew will respond to a failure of RFP A per 2-AOI-3-1.
- 3. The operating crew will respond to a control rod reed switch failure and move the rod to a position with an operable detector per technical specifications.
- 5. The operating crew will recognize and respond to fuel failure in accordance with ARPs.
- 6. The operating crew will recognize and respond to a Main Steam line break and failure of MSIVs to auto isolate per EOI-3.
- 7. The operating crew will recognize and respond to a stuck open SRV in accordance with AOI-1-1 and EOI-2.
- 8. The operating crew will respond to RPV low water level due to HPCl, RClC and CRD pump failure in accordance with EOI-1, RC/L C1, and C2.

- VI. References: The procedures used in the simulator are controlled copies and are used in development and performance of simulator scenarios. Scenarios are validated prior to use, and any procedure differences will be corrected using the procedure revision level present in the simulator. Any procedure differences noted during presentation will be corrected in the same manner. As such, it is expected that the references listed in this section need only contain the reference material which is not available in the simulator.
  - A. SOER 94-01
  - B. SOER 96-01

#### VII. Training Materials:

- A. Calculator (If required)
- B. Control Rod Insertion Sheet (If required)
- C. Stopwatch (If required)
- D. Hold Order / Caution tags (If required)
- E. Annunciator window covers (If required)
- F. Steam tables (If required)
- F. LCO/Appendix R Tracking Log
- G. Calculator
- H. Spray Cleaner
- I. Rags
- J. Markers

#### VIII. Scenario Summary

Unit two is at 95% power following turbine valve testing. One loop of RHRSW is tagged for repair of FCV 23-34. The crew will raise power with recirc. An inadvertent half scram will occur due to an instrument failure. The crew will respond per ARPs and Tech. Specs.

A RFP governor failure will run RFP A to minimum speed. The crew will reduce power per AOI-3-1 to within the capacity of the running RFPs.

A control rod reed switch will fail and the crew will reposition the rod to restore tech spec compliance.

Loose objects in the vessel cause fuel failure, main steam line leak, and Group 1 isolation signal. The MSIVs fail to auto close and must be manually closed.

When SRVs are cycled to maintain reactor pressure, PCV 1-179 will stick open resulting in decreasing RPV inventory and pressure. When an attempt to use HPCI is made a power failure occurs, resulting in system failure. When RCIC is operated, it will trip locally and radiation levels will preclude immediate restoration. 2A CRD pump trips. SLC should be lined to the test tank.

The scenario is terminated when the RPV level is depressurized due to inability to maintain level above – 185".

SURROGATE IS BRIEFED TO NOT TAKE ACTION TO CLOSE MSIVS UNLESS DIRECTED BY SRO

## A. Scenario File Summary

1. File: bat NRC/04NRC3

	MF/RF/IOR#	Description
a.	trg e1 HPCI trg e2 MODESW trg e3 RCIC trg e4 1-179	Sets triggers for HPCI start, Mode Switch out of Run RCIC start MSRV 1-179 HS to open
b.	imf sw03b imf swo3h ior zlohs235a[1] off ior zlohs2312a[1] off ior zlozi2331 off ior zlozi2337 off ior ypovfcv2334 fail_power ior ypovfcv2340 fail_power ior zaopi234 0 ior zaopi2311 0	Tagout of A2, C2 RHRSW pumps for repair of FCV 23-34.
C.	ior zdihs719d normal	RCIC trip appears to also be mechanical
d.	imf rp15a imf rp15b	Failure of MSIV closure on group 1 isolation
e.	imf hp07 (el 0)	Loss of HPCI 120V AC power
f.	imf ad01m (e4 0) 100	Stuck open SRV
g.	imf rc03 (e3 0)	RCIC trip
h.	imf rd01a (e2 0)	CRD pump trip
i.	imf rd01b	Tagout of 1B CRD pump
•	ior zlohs852a[1] off	
j.	mrf sw07 aligned	Aligns RHRSW pump to EECW
k.	imf fw13c (e2 :2)	Common mode failure of RFPs
L	imf fw13b (e2 2)	
m.	trg e2 = bat NRC/04NRC3-2	Raise fuel failure amount
n.	mrf hwo1 fast	Advance recorders

## 2. File bat NRC/04NRC3-1

	MF/RF/I	OR#		Description
a.	Imf th23	5 10:0	0	Fuel failure
b.	Imf th35a	a (none	e 6:00) 8 6:00	MSL break
	3.	File	bat NRC/04NRC3-2	
	MF/RF/I	OR#		Description
a.	mmf th23	3 20 1:	00	Fuel rupture
	4.	File	bat NRC/04NRC3-3	
	MF/RF/IO	DR#		Description
a.	dor zdihs dmf rc03		normal	Fix RCIC

#### B. Console Operator Manipulations

ELAPSED TIME	IC/MF/RF/PFK	DESCRIPTION/ACTION
Sim. Setup	rst 28	REDUCE POWER TO 95% WITH FLOW 95% power MOC
Sim. Setup	restorepref	Establishes Preference Keys
Sim. Setup	NRC/04NRC3 Setup	Verify Preference Keys
Sim. Setup	Esc	Clears Popup Window
Sim. Setup	manual	Reduce power to 95% with recirc. (NA if IC is at 95% power)
Sim. Setup	F3	See scenario summary (bat NRC/04NRC3)
Sim. Setup	Manual	Place hold order tags on FCV 23-34, 40; A2, C2 RHRSW pumps; zi-23-37/31 vlv pos. ind.; CRD pump 1B.
After power rise of 5% or when directed by lead evaluator	F4	Fails LIS 3-203B downscale (imf th30v 0)

#### ROLE PLAY: Wait 2 min, report LIS 3-203B about 0 inches in Aux. Inst. Room

5 min after Tech specs addressed or when directed by the lead evaluator	F5	imf fw30a
When the plant is stable or when directed by the lead evaluator	F6	imf rd25

ROLE PLAY: If the failed position is not noticed in 5 minutes and with lead evaluator concurrence, call as reactor engineer and report powerplex won't run a case. Ask, "Is there a problem with the rod position indication?" (The RE hasn't determined which rod is at fault at this time.)

#### B. Console Operator Manipulations

5 min after the rod is repositioned or when directed by the lead evaluator	F7	5% fuel failure and MSL leak(bat NRC/04NRC3-1)
When requested to open breaker for PCV 1-179, wait 3 min	F8	Opens breaker for PCV 1-179 (mrf ad01m out)
If requested to close breaker for PCV 1-179	F9	Closes breaker for PCV 1-179 (mrf ad01m in)
If requested to shutdown HWC	F10	Removes HWC from service (mrf OG09 shutdown)

ROLE PLAY: If asked to check RWCU Temp ATU's in Aux Inst. Room, Report 69-

835 A-D Reading 120°F.

ROLE PLAY: When asked to check on HPCI, the fuse was replaced and blew again. It smells like there is a burned transformer in the inverter. A FINN team member is here and thinks they can replace the inverter in 30 to 45 minutes if there is one in the warehouse. (updates on status may be provided as requested with progress toward success indicated)

ROLE PLAY: As the RB AUO, when directed to investigate RCIC, wait 1 minute then report back that your alarming dosimeter went off and you saw a CAM in alarm when you entered the building. You are going to get Radcon to help see if you can safely get to the RCIC Room.

B. Console Operator Manipulations

NOTE: monitor FW flow. If condensate injection is observed prior to ED, imf fw18 100 to rupture the header.

ELAPSED TIME	IC/MF/RF/PFK	DESCRIPTION/ACTION
At about 450 psig prior to ED. Leave open if ED is initiated earlier	F11	Closes PCV-1-179 (dmf ad01m)
If requested to perform Appendix 16F & 16G wait 5 min	F12	Bypasses LPCI timers (bat app16fg)
If requested to perform Appendix 7B, wait 20 min.	<shift>F1</shift>	Aligns SLC to Test Tank (bat app07b)

Terminate the scenario when the following conditions have been satisfied or upon request of the Lead Examiner.

- -RPV water level restored to + 2 to + 51" (Lead examiner may desire to terminate when satisfactory progress to achieve this standard is observed)
- -Reporting requirements have been made

Χ.	Information	to Eva	luatore:
Λ.	HHOHHAUUH	IU EVO	IDAIDIS.

- A. Ensure recorders are inking and recording and ICS is active and updating.
- B. Assign Crew Positions based on evaluation needs.

1.	SRO:	Unit Supervisor/Shift Manager	
2.	BUO:	Board Unit Operator	
3.	DUO:	Desk Unit Operator	

- C. SURROGATE DUO briefed to only close MSIVs when directed by SRO.
- D. Conduct a shift turnover with the Shift Manager and provide the Shift Manager with a copy of the Shift Turnover.
- E. Direct the shift crew to review the control board and take note of present conditions, alarms, etc.
- F. Terminate the scenario when the following conditions are satisfied or at the request of the Lead Evaluator:
  - 1. RPV level being maintained + 2 to + 51 "
  - 2. Reporting requirements have been met

#### Event 1: RAISE POWER WITH RECIRC

<u>POSN</u>	TIME	EXPECTED ACTIONS
SRO	***************************************	Notifies ODS of power ascension
SRO		Directs BUO to raise power at 10 MWE/min per 2-GOI-100-12
BUO	~~~~~~~ <sub>~~~</sub>	Raises power to rated IAW 2-GOI-100-12 and OI-68 Section 6.2 with Recirc

#### Event 2: RPS LEVEL INSTRUMENT FAILURE

POSN	TIME	EXPECTED ACTIONS
BUO	<del></del>	Announces half scram
		Reports half scram due to level from 2-XA-55-4A-2
		Verifies RPV level normal
		Refers to 2-9-4 ARP
SRO		Dispatches individual to Auxiliary Instrument Room to check LIS 3-203 B and D
SRO		Consults Tech Specs, 3.3.1.1, 3.3.6.1, 3.3.6.2, 3.3.7.1 determines Required actions are to place in trip in 12 hrs. which is met at this time.
SRO		Directs IMs to troubleshoot and repair.
SRO		Conducts briefing on loss of instrumentation.

#### Event 3: RFP A FAILURE

<u>POSN</u>	TIME	EXPECTED ACTIONS
BUO	<u></u>	Responds to RFP A ABN and RFP DISCH FLOW LOW annunciators (may attempt manual control of RFP A)
SRO		Directs power reduction per AOI-3-1
BUO		Initiates upper power runback and/or reduces recirc flow and lowers running RFP speed to <5050 rpm.
SRO	***************************************	Directs entry into AOI-68-1B
SRO	47772000000000000	Contacts Maintenance
		Notifies ODS of power reduction
BUO/ DUO		Performs 2-SR-3.3.1.1.i Directs verification that breakers 105,106, 107 are closed

#### Event 4: ROD POSITION FAILURE

<u>POSN</u>	TIME	EXPECTED ACTIONS
BUO		Identifies rod 14-35 has no position indication
SRO		Enters LCO 3.1.3 and directs insertion of the rod to an operable position indication (46)
BUO		Inserts rod 14-35 to position 46 and reports indication is restored
SRO		Exits LCO 3.1.3
SRO	VIVIENDE EN	Notifies Reactor Engineer
SRO		Directs initiation of WR

XI.	Simulator Event Guide	
	Event 5	: FUEL DAMAGE/MSL LEAK
POSN	TIME	EXPECTED ACTIONS
DUO		Announces alarms as follows: - OG Pre Trt. Rad High
		Checks off-gas flow
SRO		Notifies Chem Lab to Sample Declares NOUE (1.4-U)
DUO		Announce - Turbine Bldg area radiation high
		<ul> <li>Check instruments as follows, reporting increase in radiation RR-90-157 (OG Pretrt) RR-90-135 (MSL Rad)</li> </ul>
SRO		Evacuates Turbine Building per ARP Notifies Rad. Con.
DUO		Announce - Reactor Building Radiation High - Checks RB instruments to determine affected areas
SRO		Evacuates Reactor Building per ARP Notifies Rad Con
SRO	<del></del>	Enters 2-EOI-3 (may direct power reduction with recirc flow)

> FUEL DAMAGE/MSL LEAK (continued) Event 5:

POSN	TIME	EXPECTED ACTIONS
DUO	<del></del>	Recognize and announce Group 6 isolation on high radiation
DUO		Reports alarm "Main Steam Tunnel Temp High" and verifies on TI 1-60A
SRO	an a	Directs Core Flow Runback
SRO		Renters 2-EOI-3
SRO	<u></u>	Enters EOI-1 and Directs manual scram
BUO		Manually scrams and verifies all rods inserted
SRO		Directs BUO to carry out actions of 2-AOI-100-1
BUO		Carries out 2-AOI-100-1 actions
		<ul><li>Mode switch in S/D</li><li>Trips main turbine</li><li>Verify recirc pumps at minimum</li></ul>

- Verify recirc pumps at minimumVerifies Gp 2, 3, 6 and 8 isolations

## Event 5: FUEL DAMAGE/MSL LEAK (continued)

POSN	TIME	EXPECTED ACTIONS
DUO		Closes MSIVs prior to isolation Or Reports MSIVs not closed on MSL high temp (189 F) on panel 9-5 alarms.
SRO	-	Directs MSIV closure, if not already directed
DUO		Closes MSIVs
BUO		Announces CRD Pump Trip and loss of RFPs
SRO		Directs pressure control 800 to 1000 psig
DUO		Directs level control + 2" to + 51" using - RCIC(5D) - HPCI (5C) Controls pressure as directed using SRVs; (11A)  Attempts to control level as directed using HPCI (5C) and RCIC (5D)

## Event 6: STUCK OPEN SRV, HPCI/RCIC/CRD FAILURE

POSN	TIME	EXPECTED ACTIONS
DUO		Reports HPCI failure (120V Power Alarm)
DUO		Reports RCIC TRIP
		Reports failure of RCIC to reset
SRO		Dispatches AUO to reset RCIC
		Dispatches OS-US to replace HPCI power supply fuses
DUO		Reports PCV 1-179 failure to close after use
DUO		Cycles PCV 1-179 per AOI-1-1 and reports valve remains open
SRO		Directs actions to close PCV 1-179 per AOI-1-1 outside control room
DUO		Reports PCV 1-179 does not close
DUO		Monitors torus temperature

# Event 6: STUCK OPEN SRV, HPCI/RCIC/CRD FAILURE (continued)

POSN	TIME	EXPECTED ACTIONS
SRO		Enters EOI-2 at 95 degrees F Torus temperature
SRO	BBF 2000	Directs available RHR placed in suppression pool cooling.
DUO		Places all available Torus Cooling I/S
SRO		Requests appendices 16F and 16G
		Directs Appendix 8G
SRO		Directs Appendix 7B
BUO		Performs Appendix 7B Verifies recirc pumps trip at -45"
SRO		Directs preventing flooding vessel via Condensate system by closing RFP discharge valves
BUO	GGGGGGCYYY CO	Closes RFP discharge valves
DUO		Performs Appendix 8G
SRO	PRODUCTION OF THE PRODUCTION O	Enters C1, Directs ADS inhibited
DUO	EXCESSION VI	Inhibits ADS

# Event 6: STUCK OPEN SRV, HPCI/RCIC/CRD FAILURE (continued)

POSN	TIME	EXPECTED ACTIONS
DUO		Reports DG starts; Dispatches AUO to monitor
SRO	***************************************	When level decreases to TAF determines/verifies that sufficient injection sources are available to not enter steam cooling and continues to wait for system restoration until RPV level approaches -185" before entering C2
SRO		Directs RHR be realigned for from Torus Cooling to injection
DUO		Secures Torus Cooling and aligns RHR for injection
SRO		Directs opening 6 ADS valves
DUO		Opens 6 ADS valves as directed
SRO		Directs DUO to inject with ECCS until level is above -122" then secure systems to not exceed + 51"; Directs restoration of Torus Cooling after RWL control is established.
DUO		Operates and secures ECCS injection as directed/reestablishes Torus Cooling as directed
SRO		Directs BUO to restore level + 2" to + 51" with Condensate
BUO		Injects with Condensate to restore level +2 " to + 51"
SRO		Classifies event as Site Area Emergency (1.1-S1)

XII.	Crew Critical Tasks	
	<u>Task</u>	SAT/UNSAT
1.	Isolates MSIVs prior to 2 2-EOI-3 Area Radiations/Temperatures reaching Max Safe	
2.	Inhibits ADS	
3.	Emergency Depressurizes when below TAF and before level drops to -185"	
Add	itional Individual Critical Task	
	If SRO directs 2/3 core height override then the SRO will verify that the RHR system is fully realigned for injection when required.	

#### SCENARIO REVIEW CHECKLIST

#### SCENARIO NUMBER 04NRC3

- 6 Total Malfunctions Inserted; List: (4-8)
  - 1) LT failure
  - 2) RFP failure
  - 3) RPIS failure
  - 4) Fuel Failure
  - 5) MSL Leak with MSIV Auto Close Failure
  - 6) Stuck Open SRV
- \_2 Malfunctions That Occur After EOI Entry; List: (1-4)
  - 1) MSIV Auto Close Failure
  - 2) Stuck open SRV
- 3 Abnormal Events; List: (1-3)
  - 1) Fuel Failure
  - 2) Stuck open SRV
  - 3) RFP trip
- 2 Major Transients; List: (1-2)
  - 1) Fuel Failure
  - 2) MSL Leak
- 2 EOIs used; List: (1-3)
  - 1) EOI-3
  - 2) EOI-1
- 2 EOI Contingencies Used; List: (0-3)
  - 1) C1, C2
- 80 Run Time (minutes)
- 35 EOI Run Time (minutes); 46 % of Scenario EOI Run Time
- 3 Crew Critical Tasks (2-5)
- Yes Technical Specifications Exercised (yes/no)

#### SHIFT TURNOVER SHEET

Equipment Out of Service/LCOs FCV 23-34 repair underway, clearance has both 2A & 2C
RHR Hx inop for containment cooling. Out of service 6 hours, expected back in 12 hours. 7
day LCO entered per T.S. 3.7.1, 3.6.2.3, 3.6.2.4, 3.6.2.5. All Appendix R, T.S. LCO's have
been addressed.
1B CRD pump is tagged for breaker maintenance.
Operations/Maintenance For the Shift Raise Unit 2 to 100% power at 10MWe/min. Support
maintenance on RHRSW valve.
Power was reduced for turbine valve testing which has been completed.
Unusual Conditions/Problem Areas Thunderstorm warning in effect next 2 hours in
Limestone County.

Facility: BFN	Scenario Number:	HLT0212#5	Op-Test Number:
		Final	
Examiners: Tim Kolb Ron Aiello	<u> </u>	Operators:	
<u> </u>			

Initial Conditions: IC 45, 77% pwr

Turnover: At  $\sim$ 75% power for CBP 2B repairs and rod pattern adjustment. 1B CRD pump, FCV 74-100, and RHR 2A OOS. Withdraw rods per pull sheet, then continue power ascension with flow after CBP 2B RTS.

Malfunction	Event	Event
Number	Type*	Description
None	R-RO	Raise Power by Withdrawing rods.
	R-SRO	
rd05	C-RO	Control Rod 14-47 uncouples.
	C-SRO	_
rd04	C-RO	Rod drift out.
	C-SRO	TS for SRO
I-O	C-SRO	RWCU Spurious isolation
overrides		TS for SRO
HP07	C-SRO	HPCI Power supply failure. Restore to service after
		RPV level is below -122.5".
Fw02a	M-ALL	CBP trip and loss of unit board 2C yielding loss of
Ed08c		normal feedwater.
Th22 0.2,	M-ALL	Recirc Suction Leak/LOCA.
Pc16a,b,c		
	Number None rd05 rd04  I-O overrides HP07  Fw02a Ed08c Th22 0.2,	Number Type*  None R-RO R-SRO  rd05 C-RO C-SRO  rd04 C-RO C-SRO  I-O C-SRO  Overrides HP07 C-SRO  Fw02a Ed08c M-ALL  Th22 0.2, M-ALL

<sup>\* (</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

# SIMULATOR EVALUATION GUIDE

TITLE :	CONTROL ROD ADJUSTMENT, CONTROL ROD 14- CONTROL ROD DRIFT OUT, SPURIOUS RWCU ISC CONTROL POWER SUPPLY FAILURE, LOSS OF NO FEEDWATER, RPV RECIRC SUCTION LEAK/LOCA,	DLATION, HI DRMAL	PCI
REVISION :	0		
DATE :	April 1, 2004		
PROGRAM :	BFN Operator Training		
PREPARED BY:_	(Operations Instructor)	. \	Date
REVIEWED BY:		\	XXXIII
	( BFN Lead Examiner or Designee)		Date
REVIEWED BY:_		\	
	(Operations Training Manager or Designee)		Date
CONCURRED:	(Operations Superintendent or Designee)		Date
	(Operations Superintendent of Designee)		Date
VALIDATION BY:		\	
	(Operations US) (Required for Exam Scenarios only)		Date

NUCLEAR TRAINING					
	REVISION/USAGE LOG				
REVISION NUMBER	DESCRIPTION OF CHANGES	DATE	PAGES AFFECTE D	REVIEWED/REVISED BY	
0	INITIAL	04/01/04	All	JT	

I. Program: BFN Operator Training

II. Course: Examination Guide

III. Title: CONTROL ROD ADJUSTMENT, CONTROL ROD 14-47 UNCOUPLED,

CONTROL ROD DRIFT OUT, SPURIOUS RWCU ISOLATION, HPCI

CONTROL POWER SUPPLY FAILURE, LOSS OF NORMAL FEEDWATER, RECIRC SUCTION LEAK/LOCA, RCIC TRIP

IV. Length of Lesson: 1.5 hours

V. Examination Objectives:

#### A. Terminal Objective

- 1. Perform routine shift turnover, plant assessment and routine shift operation in accordance with BFN procedures.
- 2. Given uncertain or degrading conditions, the operating crew will use team skills to conduct proper diagnostics and make conservative operational decisions to remove equipment/unit from operation. (SOER 94-1)
- 3. Given abnormal conditions, the operating crew will place the unit in a stabilized condition per normal, abnormal, annunciator and emergency procedures.
- 4. Use step text procedural compliance (WANO).

#### B. Enabling Objectives:

- 1. The operating crew will raise power with control rods per RE instructions.
- 2. The crew will respond to an uncoupled control rod (14-47)
- The operating crew will respond to a control rod drifting out due to a stuck withdraw valve.
- 4. The operating crew will respond to Spurious RWCU isolation.
- 5. The operating crew will respond to failure of HPCI (Loss of Control Power)
- 6. The operating crew will recognize and respond to a loss of feedwater per 2-EOI-1.
- 7. The operating crew will recognize and respond to a small LOCA per 2-EOI-1 and 2, RCIC Trips.
- The crew will spray the drywell to maintain containment less than 280 degress F and pressure below PSP limit. HPCI will be returned to service and Reactor Water Level will be restored.

- VI. References: The procedures used in the simulator are controlled copies and are used in development and performance of simulator scenarios. Scenarios are validated prior to use, and any procedure differences will be corrected using the procedure revision level present in the simulator. Any procedure differences noted during presentation will be corrected in the same manner. As such, it is expected that the references listed in this section need only contain the reference material which is not available in the simulator.
  - A. SOER 94-01
  - в. SOER 96-01

#### VII. Training Materials:

- A. Calculator (If required)
- B. Control Rod Insertion Sheet (If required)
- C. Stopwatch (If required)
- D. Hold Order / Caution tags (If required)
- E. Annunciator window covers (If required)
- F. Steam tables (If required)
- F. LCO/Appendix R Tracking Log
- G. Calculator
- H. Spray Cleaner
- I. Rags
- J. Markers

# A. Scenario File Summary

## 1 File bat NRC/04NRC5

	MF/RF/IOR#	Description
a)	trg e1 MODESW	Assigns trigger
b)	trg e2 7316open	Assigns trigger
c)	imf fw02b	2B Condensate Booster Pump Trip
d)	ior zlohs0262a[1] off	Green light off for 2B Condensate Booster Pump
e)	ior zlohs0262a[2] off	White light off for 2B Condensate Booster Pump
f)	imf rh01a	2A RHR Pump tag out
g)	ior ypobkrpmparh fail_power	2A RHR Pump tag out
h)	ior zlohs745a[1] off	Green light off for 2A RHR Pump
i) j) k) l)	trg e7 withdraw1827 Imf rd04r1827 (e7 0) imf th21 (e1 2:00) .2 5:00 Imf pc16a ( e1 4:00 ) Imf pc16b ( e1 4:00 ) Imf pc16c ( e1 4:00 )	Set rod movement out trigger Drift rod 18-27 out when withdrawn Leak in Drywell Drywell Vacuum Breaker Rupture
m) n) o)	Trg e6 RCIC lor zdihs719d (e6 5:00) trip ior ypovfcv74100 fail_power_now	RCIC Trip FCV 74-100 tag out
p) q) r) s) t)	mrf hw01 fast Imf rd05r1447 Imf rd01b Ior zlohs852a [1] off Ior zdihs852a stop	Advance recorders 14-47 Rod uncoupled CRD Pump Tagout CRD Pump Tagout CRD Pump Tagout

VIII.	Console Operator Instructions			
	B.	B. Scenario File Summary		
	2.	File: bat NRC/04NRC5-1		
		MF/RF/IOR#	Description	
a)		imf ed08c (none :20)	Trips unit Board C 1 minute after file triggered	
b)		imf fw02a (none 0:25)	Trips 2A Condensate Booster pump	
c)		ior an:2xa556a19 alarm_on	Low Condensate Booster Pump Suction alarm	
d)		trg e5 false		
2.	File:	bat NRC/04NRC5-2		
		MF/RF/IOR#	Description	
a)		lor zloil64a1 off lor an:2xa555b11 alarm on	Steam Tunnel Temperature Detector Failure	
3.	File:	bat NRC/04NRC5-3		
		MF/RF/IOR#	Description	
a)		trg e3 pcisia trg e3 = dor zloil64a1 dor an:2xa555b11	Delete Steam Tunnel Temperature Detector Failure	
4.		File: bat app16fg		
		MF/RF/IOR#	Description	
a)		mrf rh14 byp	Loop I LPCI timers	
b)		mrf rh15 byp	bypassed Loop II LPCI timers bypassed	

5. File: bat NRC/04NRC5-4

		MF/RF/IOR#	Description
a)		imf rd08r1827 0 Imf rd06r1827	simulate valving out rod
b) c)		dmf rd04r1827	Delete rod drift
5.	File:	bat NRC/04NRC5-5	
		MF/RF/IOR#	Description
a)		ior zdihs691 close	Close fcv 69-1
b) c) d)		trg e4 691close trg e4 = dor zdihs691 bat rwcudeminsout	Remove override Places demins in hold
6.	File:	bat rwcudeminsin	
		MF/RF/IOR#	Description
a) b)		mrf cu01 55 1:00 mrf cu02 55 1:00	Roll in demin A Roll in demin B
7.	File:	bat rwcudeminsout	
		MF/RF/IOR#	Description
~)		mrf cu01 0 :30	Roll out demin A
a) b)		mrf cu02 0 :30	Roll out demin B

# B. Console Operator Manipulations

ELAPSED TIME	<u>PFK</u>	DESCRIPTION/ACTION
Sim. Setup	pwrst 45	75% power, SU
Sim. Setup	restorepref NRC/04NRC5	Establishes Preference Keys
Sim. Setup	setup	Verify Preference Keys
Sim. Setup	esc	Clears Popup Window
Sim. Setup	F3	Trigger e1 on mode switch (trg e1 MODESW)
Sim. Setup	manual	Trip 2B Condensate Booster Pump.
Sim. Setup	F4	See scenario summary (bat NRC/NRC5)
Sim Setup	manual	a. Tag 2A RHR Pump, b. Tag 2B CBP c. Clear alarms, d. Tag FCV 74-100
Instructor	manual	Provide turnover sheet and pull sheet to crew

# ROLE PLAY: When requested as Reactor Engineer, Notify the control room that thermal limits will be checked and permission is granted to try and recouple rod 14-47 per 2-AOI-85-2

When rod is inserted to 46	F5	dmf rd05r1447, allows recoupling control rod
While rod 18-27 is being driven to full in	F6	dmf rd04r1827; removes rod drift
If requested to valve out rod 18-27	F7	Valve out rod 18-27 (bat NRC/04NRC5-4)
OULION IN DI	MORE	FOLLOWS 1

B. Console Operator Manipulations

F8

ROLE PLAY: When requested as Reactor Engineer, Reactor Engineer will work with the System Engineer to get a plan together to deal with the inserted rod. Maintain power until the new pull sheet is ready.

2 minutes after RE has been consulted for rod 18-27

RWCU isolation
Bat NRC-04NRC5-5

ROLE PLAY: 5 minutes after Call as IM supervisor and report the crew installing a mod for improving MOVATS test capabilities accidentally landed a new lead being installed in the FCV-69-1 close circuit on a hot terminal. The circuit is restored to normal and work is stopped.

ROLE PLAY: Act as RB AUO and report demins properly in hold and you are raising flow when requested to place them in service.

When ready to restore demins to

F9

Places RWCU demins in service

service

**PCIS** 

( bat rwcudeminsin)

2 minutes after tech specs addressed for

F10

Imf hp07 (Inserts HPCI Control Power

Failure)

2 minutes after tech

specs addressed for

HPCI and maintenance requested to troubleshoot

F11

Bat NRC/04NRC5-1

(Loss of Unit Board and Feedwater)

#### B. Console Operator Manipulations

# ROLE PLAY: If Requested, AFTER RWL drops below -122", Maintenance notifies CR that HPCI Control Power Fuses have been replaced

After RWL drops below, -122" or per Lead Instructor	F12	Dmf hp07 ( Deletes HPCI loss of Control Power )
When asked to perform appendix 8e, wait 5 minutes	< shift> F1	Bat app08e
If asked to execute Appendix 16F and 16G wait 5 minutes	<shift> F2</shift>	LPCI timers bypassed (bat app16fg)
If requested to pump down the torus	<shift> F3 or <shift> F4</shift></shift>	Lines up and starts RHR drain pump A ( bat app18rhra) Lines up and starts RHR drain pump B ( bat app18rhrb)
When requested wait 25 minutes	<shift> F5</shift>	Bat appo7b ( Aligns SLC for injection )

Terminate the scenario when the following conditions are satisfied or when requested by Lead Evaluator:

Drywell pressure is below PSP Reactor level stable between + 2" and + 51"

#### IX. Scenario Summary

The unit is operating at 70% power. The 2B Condensate Booster Pump is out of service for repair of an oil leak. The 2A RHR pump is out of service for preventive maintenance. Currently in a 7 day LCO via Tech Spec 3.5.1. FCV 74-100 is tagged out for maintenance.

The crew will commence to raise power by pulling control rods, Control Rod 14-47 will uncouple and the crew will respond to an uncoupled rod per 2-AOI-85-2, Control rod 18-27 will drift out requiring the crew to fully insert it per 2-AOI-85-6.

The crew will respond to a spurious RWCU System isolation and restore the system to service.

A spurious low suction pressure signal will cause a loss of condensate booster pump 2A. Condensate booster pump 2C will trip due to a overcurrent lockout of 2C unit board. A loss of normal feedwater will occur and the reactor will scram.

HPCI fails to initiate (Loss of Control Power), A small RPV suction leak occurs, requiring venting, suppression pool sprays, and drywell sprays per EOI-2. Three drywell vacuum breakers rupture causing a partial loss of pressure suppression function. RCIC trips, Maintenance activities allow restoration of HPCI and Reactor water Level is restored before Rx Level reaches -185"

Χ.	Floor	Instructor	Instructions
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A.	Assign crew positions (assign positions based on evaluation requirements for
	personnel).

1.	SRO	: Unit Supervisor/Shift	
		Manager	
2.	BUO	: Board Unit Operator	
3.	DUO	: Desk Unit Operator	147

- B. Review the shift briefing information with the operating crew. Provide SRO with a copy of Shift Turnover Sheet, Reactivity Plan, and pull sheet for scenario.
- C. Direct the shift crew to review the control boards and take note of present conditions, alarms, etc.
- D. Ensure recorders are inking and recording and ICS is active and updating. Note any deficiencies during shift briefing.
- E. Terminate the scenario when the following conditions are satisfied or upon request of Lead Examiner:
  - 1. Drywell pressure is below PSP
  - 2. RPV level at + 2" to + 51"

Event 1:	RAISE	POWER	USING	CONTROL	RODS

POSN	TIME	EXPECTED ACTIONS
CREW		Accepts shift turnover
SRO		Directs BUO to withdraw rods per movement sheet
BUO		Withdraws rods per movement sheet
DUO		Peer Checks rod withdrawals

SRO

	Event 2:	CONTROL ROD !4-47 UNCOUPLED
POSN	TIME	EXPECTED ACTIONS
CREW	<del></del>	Recognizes symptoms of an uncoupled rod
SRO		Directs BUO to respond to uncoupled rod per 2-AOI-85-2
SRO		Notifies Reactor Engineer of uncoupled rod per 2-AOI-85-2 and requests permission to proceed with re-coupling rod 14-47
BUO		Performs actions to recouple rod per 2-AOI-85-2, section 4.2.3
BUO		Performs coupling check of rod 14-47

Determines rod is restored to Operable

	Event 3:	ROD DRIFT OUT
POSN	TIME	EXPECTED ACTIONS
BUO		Responds to rod drift annunciator; - identifies rod; - informs SRO; - selects rod and applies insert signal per 2-AOI-85-6 (CR 18-27)
SRO		Enters 2-AOI-85-6 and directs actions
		Checks TS 3.1.3.C (not INOP until maintenance begins)
SRO		Consults with reactor engineer (and system engineer)
BUO		drives rod to 00 releases insert signal and determines rod settles at 00

Event 4:

DUO \_\_\_\_\_

DUO \_\_\_\_

**RWCU ISOLATION** 

**Notifies Chemistry** 

Notifies Rx. Engineer

POSN	TIME	EXPECTED ACTIONS
Crew		Recognizes RWCU Isolation, responds per the ARP
SRO		Enters 2-AOI-64-2a
SRO		Contacts Rx Engineer for heat balance
SRO		Notify Chemistry and RadCon
SRO		Evaluate Tech. Spec. (TRM TSR 3.4.1) Chemistry, sampling required (If not returned to service)
SRO		Directs restoration of system after cause is determined
BUO		Restores RWCU per 2-OI-69 (optional) -Verifies RWCU Filters are in Hold -Opens 2-FCV-69-1 -Opens 2-FCV-69-8 -Adjust 2-FCV-69-12 for pump startup -Start one RWCU pump and open 2-FCV-69-12 to achieve > 56 gpm -Start the second RWCU pump and attain ~300 gpm flow -coordinate with RB AUO and place demins in service

	Event 5:	LOSS OF 120VAC HPCI CONTROL POWER
POSN	TIME	EXPECTED ACTIONS
DUO		Announces Loss of HPCI Control Power Annunciator on Panel 9-3
SRO	AMDIGUESTIE	Dispatches field personnel to investigate and troubleshoot the loss of Control Power per ARP
SRO		Declares HPCI Inop per TS 3.5.1.D, 72 hour LCO due to inop RHR A.

	Event 6:	LOSS OF FEEDWATER
POSN	TIME	EXPECTED ACTIONS
DUO		Announces "Cnds Bstr Pump "2A" Suct Press Low" Alarm
DUO		Announces Condensate Booster Pump 2A trip
Crew		Identifies loss of 2C unit board
BUO		Announces loss of RFP's
SRO	***************************************	Directs manual scram
BUO		Inserts a manual scram; Places mode switch in shutdown verifies all rods in makes scram report
DUO		Trips main turbine

	Event 6:	LOSS OF FEEDWATER (continued)
POSN	TIME	EXPECTED ACTIONS
SRO	***************************************	Enters EOI-1 on low level and directs: -Verifications of group isolations and initiations -entry into AOI-100-1 -verifies Turbine Bypass valves controlling pressure
DUO	erwin nor expositional way	Verifies Group isolations (2, 3, 6, & 8)
DUO		Starts/verifies start RCIC
BUO	<u> </u>	Inserts SRMs & IRMs and follows power down
		Verifies Recirc pumps run back/trip
		Shutdown H <sub>2</sub> water chemistry
SRO		Dispatches US/AUO to check status of 2C unit Board

# Event 7: RPV Recirc Suction Leak/LOCA

POSN	TIME	EXPECTED ACTIONS	
Crew		Identifies drywell pressure/temperature rise	
SRO		Enters EOI-2 and re-enters EOI-1 when drywell pressure exceeds 2.45 psig or orus level.	
DUO	40*-	Verifies Diesel generators start	
DUO		Directs AUO to monitor D/G	
SRO		<ul> <li>Directs the following:</li> <li>All available drywell cooling in service</li> <li>Venting per App 12</li> <li>H<sub>2</sub>/O<sub>2</sub> Analyzers placed in service</li> <li>Directs Appendix 8G</li> </ul>	
DUO		Verifies all drywell blowers in service	
DUO		Vents per Appendix 12	
DUO		Places H <sub>2</sub> /O <sub>2</sub> Analyzers in service using keylock bypass switches	
DUO	***************************************	Performs App 8G	

# Event 7: RPV Recirc Suction Leak/LOCA (continued)

POSN	TIME	EXPECTED ACTIONS	
SRO		Directs Appendix 5B from CRD	
BUO		Performs Appendix 5B	
DUO	***************************************	Monitors containment parameters and reports rate of rise of suppression chamber pressure and drywell temperature	
SRO		Determines venting and plant cooldown will not maintain suppression chamber pressure <12 psig and:	
SRO		Directs Torus Sprays be placed in service per Appendix 17C	
DUO		Places Torus sprays in service per Appendix 17C	
SRO	<del></del>	When PSC pressure exceeds 12 psig or if determines drywell temperature to exceed 280 degrees F	
		<ul> <li>verifies in Safe Region of DW spray curve</li> <li>verifies SP level &lt;18 feet</li> </ul>	
SRO		Directs drywell blowers removed from service	
SRO		Directs drywell sprays initiated	
SRO	***************************************	Directs drywell sprays stopped before drywell pressure drops below 0 psig	

	Event 6:	RPV Recirc Suction Leak/LOCA (continued)	
POSN	TIME	EXPECTED ACTIONS	
SRO		Directs LPCI timers bypassed per Appendix 16F and 16G	
DUO		Trips/verifies Recirc pumps tripped	
DUO		Removes drywell blowers from service or reports trips of blowers	
DUO		Initiates drywell sprays	
Crew		Continues to monitor Rx Level and containment parameters	
DUO		Recognizes loss of RCIC	
SRO		Dispatches AUOs/OSUS to investigate loss of RCIC	

Event 6: RPV Recirc Suction Leak/LOCA (continued)

POSN	TIME	EXPECTED ACTIONS	
SRO		Directs DUO to utilize HPCI to restore level when notified by Maintenance of the successful restoration of 120 VAC Control Power	
Crew	172000A	Monitors PC temp and pressure	
SRO		Direct Drywell sprays removed before 0 psig Drywell Pressure	
DUO	17000000	Secures Drywell Sprays	
SRO		Directs level be maintained above -162 using available systems	
DUO		Maintains level above -162 using Core Spray, LPCI, & HPCI (If inservice)	
SRO		Directs Drywell spray again (if required)	
SRO	***************************************	Classifies event as an ALERT (2.1-A)	

#### XII. Crew Critical Tasks

	TASK	SAT/UNSAT
1.	Maintain RPV water level above TAF	
2.	Maintain Drywell pressure below PSP	

#### SCENARIO REVIEW CHECKLIST

#### SCENARIO NUMBER 2004NRC5

- 8 Total Malfunctions Inserted; List: (4-8)
  - 1) Loss of 2A Condensate Booster Pump
  - 2) Coolant Leak in Drywell
  - 3) Control Rod Uncoupled
  - 4) Drywell Vacuum Breaker Failures
  - 5) Control Rod drifts out
  - 6) RWCU Isolation valve closure
  - 7) HPCI Power supply failure
  - 8) RCIC Trip
- 3 Malfunctions That Occur After EOI Entry; List: (1-4)
  - 1) Steam Leak in Drywell
  - 2) RCIC Trip
  - 3) Drywell Vacuum Breaker Failure
- 3 Abnormal Events; List: (1-3)
  - 1) Control Rod Uncoupledt
  - 2 Control Rod Drifts Out
  - 3) RWCU isolation
- 2 Major Transients; List: (1-2)
  - 1) Loss of Normal Feedwater
  - 2) Drywell Leak
- 2 EOIs used; List: (1-3)
  - 1) EOI-1
  - 2) EOI-2
- 1 EOI Contingencies Used; List: (0-3)
  - 1) C1
- 80 Run Time (minutes)
- 35 EOI Run Time (minutes); 25 % of Scenario EOI Run Time
- 2 Crew Critical Tasks (2-5)
- Yes Technical Specifications Exercised (yes/no)

#### SHIFT TURNOVER SHEET

Equipment Out of Service/LCOs <u>Unit at 75% power due to 2B Condensate Booster Pump out</u>
of service for oil lea k. 2B Condensate pump secured per GOI. CRD Pump 1B tagged out for
Breaker work
2-FCV-74-100 tagged out for maintenance on the actuator, valve has a mechanical locking
device installed and is closed. Actuator is removed.
2A RHR pump was found 2 hours ago with no oil in the upper sight glass. Dual Unit matrix
addressed and a priority 2 work order has been initiated. The system engineer is evaluating
compensatory measures. A 7 day LCO (TS 3.5.1) and 30 day LCO (TS 3.6.2 ,3, 4, & 5) has
been entered and the pump has been tagged.
Mech maint estimates 3 hours to repair a leaking oil fitting which was found damaged.
Operations/Maintenance For the Shift: withdraw rods per RE instructions to establish rod
pattern. Do not exceed 80% power until 2B Condensate Booster Pump is repaired (expected
late this shift).
When 2B CBP is restored, return to rated power.
Unusual Conditions/Problem Areas Reactor Engineer Expects Xenon to build-in over next few
hours.