

September 2, 2004

MEMORANDUM TO: Daniel S. Collins, Acting Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Scott P. Wall, Project Manager, Section 2 /RA/
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: SEABROOK STATION, DRAFT REQUEST FOR ADDITIONAL
INFORMATION (TAC NO. MC2364)

By letter dated August 18, 2004, the Nuclear Regulatory Commission (NRC) staff issued a request for additional information (RAI) regarding the FPL Energy Seabrook, LLC (FPLE or the licensee), application to increase the licensed thermal power level at Seabrook Station, Unit No. 1.

The NRC staff determined that additional information was needed. The attached RAIs were transmitted on September 1, 2004, to Mr. Michael O'Keefe (FPLE) as additional RAIs to the August 18, 2004, letter.

Docket No. 50-443

Attachment: RAI

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Attachment: RAI

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING PROPOSED STRETCH POWER UPRATE (SPU) AMENDMENT
SEABROOK STATION, UNIT NO. 1 (SEABROOK)
DOCKET NO. 50-443

Containment Assessment

96. In support of LAR Section 6.4.1, "Loss of Coolant Accident (LOCA) Mass and Energy Release," justify the use of equipment qualification temperatures for the acceptance criterion for containment LOCA calculations. Why is this acceptable to demonstrate structural adequacy?
97. In Section 6.4.1.1.7, "Acceptance Criteria for Analyses," FPLE states that the criteria for sources of heat for the LOCA mass and energy release calculations is stated as Appendix K Paragraph I.A. However, the decay heat is stated in 6.4.1.1.8, "Mass and Energy Release Data," as being calculated from Reference 6.4-5, "American National Standard for Decay Heat Power in Light Water Reactors." These are different models. Please explain the inconsistency.
98. In Section 6.4.4.2, "Input Parameters and Assumptions," and Section 6.4.4.4, "Results," FPLE gives Reference 6.4-1, "Westinghouse LOCA Mass and Energy Release Model for Containment Design -March 1979 Version," as a reference for main steam line break mass and energy release guidance. Reference 6.4-1 refers to LOCAs. Verify that this is the correct reference.
99. State the version of the GOTHIC computer code was used for the Seabrook containment analysis. Verify that GOTHIC was used consistent with the NRC safety evaluation report for Kewaunee dated September 29, 2003.