PSEG Nuclear LLC
P.O. Box 236, Hancocks Bridge, New Jersey 08038-0236

AUG 2 3 2004

PSEG
Nuclear LLC

LR-N04-0365
LCR H02-002

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

**RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION REGARDING REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS TO SUPPORT REMOVAL OF THE REACTOR VESSEL HEAD SPRAY PIPING
HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354**

Reference: Letter LR-N04-0310, Response To Request For Additional Information Regarding Request For Change To Technical Specifications To Support Removal Of The Reactor Vessel Head Spray Piping, dated July 15, 2004.

On July 15, 2004, PSEG Nuclear LLC (PSEG) submitted a response to the referenced request for additional information in support removal of the Reactor Head Spray System. Mr. D. Collins, Hope Creek Project Manager called on July 16, 2004 to request clarification of our response on General Design Criteria (GDC) 14. Subsequently, he also requested clarification with respect to our response to GDC 53. This information was provided verbally. PSEG is docketing the information as Attachment 1 to this letter.

If you have any questions or require additional information, please contact Mr. Michael Mosier at (856) 339-5434.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 23 August 2004

John Carlin
Vice President – Nuclear Assessments

Attachment

A001

C: Regional Administrator – NRC Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. D. Collins, Project Manager – Hope Creek
U.S. Nuclear Regulatory Commission
Mail Stop 08C2
Washington, DC 20555-0001

USNRC Senior Resident Inspector – Hope Creek (X24)

Mr. K. Tosch, Manager IV
Bureau of Nuclear Engineering
PO Box 415
Trenton, New Jersey 08625

Attachment 1

Response to NRC Request For Additional Information

Question 1

In your response to General Design Criteria 14, Reactor Pressure Coolant Pressure Boundary, PSEG indicated that a Reactor Pressure Vessel (RPV) hydrostatic test would be used to test the blind flange installed on nozzle N6A. Provide clarification as to whether this is an ASME over pressure hydrostatic test or a leakage test.

PSEG Response

The test to be performed is a leakage test. HC.OP-IS.ZZ-0001, "Inservice System Leakage Test of the Reactor Coolant Pressure Boundary", is performed at normal operating temperature and pressure.

Question 2

In your response to General Design Criteria, Provisions for containment testing and inspection, you did not provide the design criteria for the six-inch pipe cap. Please provide the design pressure for the six-inch pipe cap.

PSEG Response

The six-inch pipe cap is a Schedule 40, Class GBB, ASME III B&PV, Section III, Class 2, 300 psig, 850 degrees F. The pipe cap weld is ASME III, Class MC for pressure retaining components. The design pressure for containment is 62 psig and the design accident value is 48.1 psig. The original pipe was designed to ASME Class 1, Schedule 80.

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