Entergy

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RBG-46303

August 24, 2004

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: Supplement to Amendment Request
Deletion of Technical Specification 3.6.4.4 Shield Building Annulus Mixing System and revision of Main Steam Isolation Valve (MSIV) Surveillance Requirement SR 3.6.1.3.10 (TAC No. MC1094)
River Bend Station, Unit 1
Docket No. 50-458
License No. NPF-47

REFERENCES:
1. Letter RBG-46183 from Entergy to USNRC, "License Amendment Request - Deletion of Technical Specification 3.6.4.4 Shield Building Annulus Mixing System and revision of Main Steam Isolation Valve (MSIV) Surveillance Requirement SR 3.6.1.3.10," dated October 21, 2003.
2. Letter RBG-46155 from Entergy to USNRC, "NRC Generic Letter 2003-01, Control Room Habitability, 60-day response," dated August 11, 2003.

Dear Sir or Madam:

By letter dated October 21, 2003 (Reference 1), Entergy Operations, Inc. (Entergy) proposed a change to the River Bend Station (RBS) Technical Specifications (TSs) to delete Technical Specification 3.6.4.4, "Shield Building Annulus Mixing System", and revise Main Steam Isolation Valve (MSIV) Surveillance Requirement SR 3.6.1.3.10. In addition, by letter dated August 11, 2003 (Reference 2), Entergy provided the RBS 60-day response to Generic Letter (GL) 2003-01, *Control Room Habitability.* In response to the GL, Entergy committed to complete certain actions and submit a written report to the NRC summarizing the results no later than January 31, 2005. One of the actions was to complete control room envelope (CRE) in-leakage testing to confirm that habitability requirements were met. This testing was completed on May 3, 2004. During the review of the proposed TS changes submitted by Reference 1, members of the NRC staff determined that the results of the completed CRE in-leakage tests were needed to complete their review. The purpose of this letter is to provide the results of the completed CRE tests.

Two CRE tests were performed using the methods of American Society of Testing and Materials (ASTM) Standard E741, *Standard Test Method for Determining Air Change in a Single Zone by Means of a Tracer Gas Dilution.* The first test was performed with the Control Room Fresh Air (CRFA) system isolated and one division of the CRFA system operating in

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the emergency filtration mode. The second test was also performed with the CRFA system isolated and the other division of the CRFA system operating in the emergency filtration mode. There were no special configurations or alignments in the adjacent areas for either test. The results of the tests successfully confirmed that the current RBS design basis unfiltered in-leakage assumption of 300 cfm was met. The tests demonstrated that the actual unfiltered in-leakage was statistically zero cfm. The tests were performed by Lagus Applied Technology in conjunction with Nuclear Consulting Services.

There are no technical changes proposed to the amendment request. The original no significant hazards consideration included in Reference 1 is not affected by any information contained in this supplemental letter. There are no new commitments contained in this letter.

If you have any questions or require additional information, please contact Ron Byrd at 601-368-5792.

I declare under penalty of perjury that the foregoing is true and correct. Executed on August 24, 2004.

Sincerely,

R. J. King
Director- NSA

RJK/RWB

cc: Dr. Bruce S. Mallett
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