

August 30, 2004

LICENSEE: Indiana Michigan Power Company
FACILITY: Donald C. Cook Nuclear Plant, Units 1 and 2
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON AUGUST 12, 2004,
BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION (NRC) AND
INDIANA MICHIGAN POWER COMPANY (I&M) REPRESENTATIVES
CONCERNING REQUESTS FOR ADDITIONAL INFORMATION ON
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL
APPLICATION (TAC NOS. MC1202 AND MC1203)

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Indiana Michigan Power Company (the applicant) held a telephone conference call on August 12, 2004, to discuss requests for additional information (RAIs) concerning the Donald C. Cook Nuclear Plant (CNP) license renewal application (LRA). The conference call consisted of discussions on I&M's responses to previously submitted RAIs with which the staff had additional questions. During the conference call, the applicant also provided answers to clarification questions asked by the staff during a July 8, 2004, conference call concerning the LRA.

On the basis of the discussions, the applicant was able to better understand the staff's RAI. The conference call was also useful in clarifying the staff's questions. No staff decisions were made during the meeting.

Enclosure 1 provides a listing of the telephone conference participants. Enclosure 2 contains the RAIs discussed with the applicant and the questions for which clarification was requested, including a brief description on the status of the item. Enclosure 3 contains documentation used to support the applicant's responses. The applicant has had an opportunity to comment on this summary.

/RA/

Jonathan Rowley, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

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SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON AUGUST 12, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION (NRC) AND INDIANA MICHIGAN POWER COMPANY (I&M) REPRESENTATIVES CONCERNING REQUESTS FOR ADDITIONAL INFORMATION ON DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION (TAC NOS. MC1202 AND MC1203)

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/RA/

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Docket Nos.: 50-315 and 50-316

Enclosures: As stated

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DRAFT REQUESTS FOR ADDITIONAL INFORMATION AUGUST 12, 2004**

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**REQUESTS FOR ADDITIONAL INFORMATION (RAIs) DISCUSSED FOR
DONALD C. COOK (CNP), UNITS 1 AND 2, LICENSE RENEWAL
DURING AUGUST 12, 2004 TELEPHONE CONFERENCE**

RAI 4.3.1-1

Section 4.3.1 of the LRA discusses the fatigue evaluation of the Unit 1 auxiliary spray line that was performed in response to NRC Bulletin 88-08, "Thermal Stresses in Piping Connected to Reactor Coolant Systems." The LRA indicates that this fatigue evaluation is contained in WCAP-14070, "Evaluation of Cook Units 1 and 2 Auxiliary Spray Piping per NRC Bulletin 88-08," July 1994. Provide a copy of WCAP-14070.

I&M Response to RAI 4.3.1-1:

As stated in June 16, 2004 letter AEP:NRC:4034-08.

Status: The first discussion on the response to this RAI was held on July 8, 2004 (NRC letter dated July 30, 2004). During the August 12, 2004 conference call, the applicant stated the need to consult with Westinghouse for more information to address this question. The applicant is to supplement its response after consultation with Westinghouse.

RAI 4.3.3-1

Section 4.3.3 of the LRA discusses I&M's evaluation of the impact of the reactor water environment on the fatigue life of components. The discussion references the fatigue sensitive component locations for an early vintage Westinghouse plant identified in NUREG/CR-6260, "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components." The LRA indicates that the design usage factors provided in Table 5-98 of NUREG/CR-6260 were used for the evaluation of the charging nozzle, safety injection nozzle and RHR [residual heat removal] tee. The design usage factors were based on an evaluation of the Turkey Point facility, including a plant specific evaluation of the RHR piping and detailed finite element analyses of the charging and safety injection nozzles. Discuss the applicability of these analyses to the Cook facility. The discussion should include a comparison of piping sizes and thicknesses, including the design of the thermal sleeves between Cook and Turkey Point. The discussion should also include a comparison of the number and type of design transients cycles between Cook and Turkey Point.

I&M Response to RAI 4.3.3-1:

As stated in June 16, 2004 letter AEP:NRC:4034-08.

Status: The first discussion on the response to this RAI was held on July 8, 2004 (NRC letter dated July 30, 2004). During the August 12, 2004 conference call, the applicant stated the need to consult with Westinghouse for more information to address this question. The applicant is to supplement its response after consultation with Westinghouse.

RAI 4.6.1-1

Section 4.6.1 of the LRA discusses the evaluation of the containment liner. The LRA indicates that the liner was evaluated in 1999 after the discovery of localized thinning of the liner. Indicate the amount and extent of the localized liner thinning. Describe how the fatigue evaluation of the locally thinned area was performed.

I&M Response to RAI 4.6.1-1:

As stated in June 16, 2004 letter AEP:NRC:4034-08.

Status: The first discussion on the response to this RAI was held on July 8, 2004 (NRC letter dated July 30, 2004). During the August 12, 2004 conference call, the staff requested the applicant to provide reference material that had previously been approved by the NRC..

Request for Clarification:

4.3.1-0 (S1)

The staff requests the applicant to provide clarification regarding the statement that "I&M has modified the plant design and operations to preclude feedwater nozzle cracking from being a concern" in CNP LRA Table 4.3-1.

Status: I&M provided a letter (AEP:NRC:00305 dated November 26, 1979, see Enclosure 3) to the staff that was previously approved by the NRC which contained information on the modification of the main feedwater system at the Cook Plant. The applicant indicated the requested clarification information was contained in the "Design Modifications" section of attachment 1.

4.3.1-0 (S2)

The staff requests the applicant to provide clarification regarding why the transient, "120 cycles of secondary to primary leak tests for the Mode 51R replacement steam generator," is listed in Table 4.1-10 of the UFSAR but listed in Table 4.3-1 of the CNP LRA.

Status: I&M provided a response to this question in an email (see Enclosure 3).

4.3.1-0 (S3)

The staff requests the applicant to provide clarification regarding why "Reactor Vessel Internals" is listed in Section 4.3 but not in Section 4.3.1 of the CNP LRA.

Status: I&M clarified that the Reactor Vessel Internals did not require fatigue cumulative usage factors (CUFs) calculations. Thus, they were not included in Section 4.3.1.

Donald C. Cook Nuclear Plant, Units 1 and 2

cc:

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