

August 23, 2004

Mr. R. T. Ridenoure  
Division Manager - Nuclear Operations  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
Post Office Box 550  
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 – REQUEST FOR ADDITIONAL  
INFORMATION (TAC NO. MC3217)

Dear Mr. Ridenoure:

By letter dated May 21, 2004, Omaha Public Power District (OPPD) submitted an amendment request for the Fort Calhoun Station, Unit 1. OPPD proposed to modify the Basis of Technical Specification 2.4, "Containment Cooling," to allow containment spray pumps to be secured during a loss-of-coolant accident to minimize the potential for containment sump clogging when certain conditions are met. The staff has completed its preliminary review of this submittal and has determined it needs additional information to complete the review. Our request for additional information is enclosed. This request was discussed with Thomas Matthews of your staff and it was agreed that a response would be provided within 30 days of receipt of this letter.

Sincerely,

***/RA/***

Alan B. Wang, Project Manager, Section 2  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: Request for Additional Information

cc w/encl: See next page

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**REQUEST FOR ADDITIONAL INFORMATION**  
**TECHNICAL SPECIFICATION 2.4 BASIS CHANGE**  
**OMAHA PUBLIC POWER DISTRICT**  
**FORT CALHOUN STATION, UNIT NO. 1**  
**DOCKET NO. 50-285**

NRC Bulletin 2003-01, "Potential Impact of Debris Blockage on Emergency Sump Recirculation at Pressurized Water Reactors," required that operators of pressurized water reactors (PWRs) state that the emergency core cooling systems (ECCS) and the containment spray (CS) recirculation functions meet applicable regulatory requirements with respect to adverse post-accident debris blockage or describe interim compensatory measures to reduce the risk associated with the potentially degraded or non-conforming ECCS and CS recirculation functions. By letter dated May 21, 2004, Omaha Public Power District (OPPD) submitted an amendment request for the Fort Calhoun Station, Unit 1 (FCS) as part of its commitment made to this bulletin. OPPD proposed to modify the Basis of Technical Specification 2.4, "Containment Cooling," to allow CS pumps to be secured during a loss-of-coolant accident (LOCA) to minimize the potential for containment sump clogging when certain conditions are met. The staff has completed its preliminary review of this submittal and has determined it needs the following additional information to complete the review.

1. Describe the steps necessary to re-start a CS pump that has been stopped after starting on an automatic actuation signal. Assess the likelihood that it will not re-start when required.
2. Verify that the large break LOCA peak cladding calculation is done in accordance with Section 6.2.1.5, "Minimum Containment Pressure Analysis for Emergency Core Cooling System Performance Capability Studies," of the Standard Review Plan, in particular, Branch Technical Position CSB 6-1, B.2, which states that all engineered safety feature containment heat removal systems operate at maximum heat removal capacity. Does the analysis assume operation of all three CS pumps and both trains of containment fan coolers? Where is this documented?
3. Verify that the design basis calculation of radiological dose following a large break LOCA credits one CS pump. Where is this documented?
4. In addition to the successful operation of the safety injection system, describe other indications of possible inadequate core cooling that the operator would use during a large break LOCA.