

August 19, 2004

LICENSEE: Nuclear Management Company, LLC
FACILITY: Point Beach Nuclear Plant, Units 1 and 2
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE HELD ON JULY 15, 2004,
BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND
NUCLEAR MANAGEMENT COMPANY, LLC, CONCERNING DRAFT
REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE POINT
BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL
APPLICATION

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on July 15, 2004, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant License Renewal Application. The telephone conference was useful in clarifying the intent of the staff's D-RAIs.

Enclosure 1 provides a list of the telephone conference participants. Enclosure 2 contains a listing of the D-RAIs discussed with the applicant, including a brief description on the status of the items.

The applicant has had an opportunity to comment on this summary.

/RA/ Samson Lee for

Michael J. Morgan, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-266 and 50-301

Enclosures: As stated

cc w/encls: See next page

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ADAMS Accession No.: ML042320653

DOCUMENT NAME: C:\ORPCheckout\FileNET\ML042320653.wpd

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TO DISCUSS THE POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
LICENSE RENEWAL APPLICATION
JULY 15, 2004**

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Brian Moy
Alvin Chiu
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Nuclear Management Company, LLC (NMC)
NMC
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DRAFT REQUESTS FOR ADDITIONAL INFORMATION (D-RAI)
POINT BEACH NUCLEAR PLANT, UNITS 1 & 2
LICENSE RENEWAL APPLICATION
JULY 15, 2004

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Nuclear Management Company, LLC (NMC) held a telephone conference call on July 15, 2004, to discuss and clarify the staff's draft requests for additional information (D-RAIs) concerning the Point Beach Nuclear Plant, Units 1 and 2, license renewal applications (LRA). The following D-RAIs were discussed during the telephone conference call.

Section 3.2 Engineered Safety Features

RAI 3.2-1

In LRA Tables 3.2.2-1, 3.2.2-2, and 3.2.2-3, stainless steel heat exchangers, with heat transfer as the intended function, are identified with only internal environments and associated aging effects. The external environment is listed as N/A. The applicant is requested to explain why for these components the external environment and associated aging effects need not be identified. The applicant is also requested to address the similar question for the stainless steel heat exchangers in Tables 3.2.2-1 and 3.2.2-2, which have pressure boundary as their intended function.

Discussion:

The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

RAI 3.2-2

In LRA Tables 3.2.2-1 and 3.2.2-2, carbon/low alloy steel tank components are identified with only external environments and the associated aging effects. The internal environment is listed as N/A. On the other hand, stainless steel tank components are identified with internal environments with the associated aging effects. The applicant is requested to explain the material construction of the tank, and the environments to which it is exposed, to support the information provided in the tables.

Discussion:

The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

Section 3.4 Steam and Power Conversion System

RAI 3.4-1

In LRA Table 3.4.2-3, stainless steel heat exchangers, with either heat transfer or pressure boundary as the intended function, are identified with only internal environments and associated aging effects. The external environment is listed as N/A. The applicant is requested to explain

why for these components the external environment and associated aging effects need not be identified.

Discussion:

The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

RAI 3.4-2

In the first quarter of 2003, Point Beach Nuclear Plant entered the Multiple/Repetitive Degraded Cornerstone Column (Column IV) of the Action Matrix of NRC Inspection Manual Chapter 0305, "Operating Reactor Assessment Program," as a result of a high significance (Red) inspection finding involving the potential for a common mode failure of the auxiliary feedwater system (AFW) following a loss of the instrument air system. A second Red inspection finding (Yellow for Unit 1 and Red for Unit 2) was subsequently identified which involved the potential common mode failure of the AFW pumps due to plugging of the recirculation line pressure reduction orifices. In view of the aforementioned inspection findings and the subsequent corrective actions for the two AFW issues, the applicant is requested to address the following questions in the context of the license renewal application:

- (a) Clarify whether the instrument air system and the recirculation line pressure reduction orifices are in scope for the LRA evaluation, and that appropriate AMPs have been identified to manage the components.
- (b) For aging management evaluation as summarized in LRA Table 3.4.2-3 for the auxiliary feedwater system, discuss any impact on the scoping of mechanical components in the auxiliary feedwater system, and the AMR performed for the affected components, as a result of the recent physical modifications made to resolve the auxiliary feedwater pump room Appendix R issues.

Discussion:

The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.