August 18, 2004

Mr. Edward J. Weinkam Director, Regulatory Services Nuclear Management Company, LLC 700 First Street Hudson, WI 54016

SUBJECT: DUANE ARNOLD ENERGY CENTER, KEWAUNEE NUCLEAR POWER PLANT, MONTICELLO NUCLEAR GENERATING PLANT, PALISADES NUCLEAR PLANT, POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 — REQUEST FOR ADDITIONAL INFORMATION RE: REQUEST FOR APPROVAL OF NUCLEAR MANAGEMENT COMPANY QUALITY ASSURANCE TOPICAL REPORT (TAC NOS. MB7157, MC1309, MC1310, MC1311, MC1312, MC1313, MC1314, MC1315, AND MC1316)

Dear Mr. Weinkam:

By application to the U.S. Nuclear Regulatory Commission (NRC) dated October 31, 2003, as supplemented July 23, 2004, Nuclear Management Company, LLC (NMC) requested approval of the NMC Quality Assurance Topical Report. The NRC staff is reviewing your request and finds that additional information is needed as shown in the enclosed request for additional information (RAI). The enclosed RAI questions relate to Enclosures 1 through 5 of your submittal.

The NRC staff discussed the enclosed RAI with your staff and on August 4, 2004, you agreed to respond within 30 days of receipt of the RAI. Please contact me at (301) 415-3243 if you have questions.

Sincerely,

#### /RA/

David P. Beaulieu, Project Manager, Section 1 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket Nos. 50-331, 50-305, 50-263, 50-255, 50-266, 50-301, 50-282, and 50-306

Enclosure: Request for Additional Information

cc w/encl: See next page

Mr. Edward J. Weinkam Director, Regulatory Services Nuclear Management Company, LLC 700 First Street Hudson, WI 54016

SUBJECT: DUANE ARNOLD ENERGY CENTER, KEWAUNEE NUCLEAR POWER PLANT, MONTICELLO NUCLEAR GENERATING PLANT, PALISADES NUCLEAR PLANT, POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 — REQUEST FOR ADDITIONAL INFORMATION RE: REQUEST FOR APPROVAL OF NUCLEAR MANAGEMENT COMPANY QUALITY ASSURANCE TOPICAL REPORT (TAC NOS. MB7157, MC1309, MC1310, MC1311, MC1312, MC1313, MC1314, MC1315, AND MC1316)

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# **REQUEST FOR ADDITIONAL INFORMATION**

## **REQUEST FOR APPROVAL OF**

# NUCLEAR MANAGEMENT COMPANY QUALITY ASSURANCE TOPICAL REPORT

# NUCLEAR MANAGEMENT COMPANY, LLC

# DUANE ARNOLD ENERGY CENTER

# KEWAUNEE NUCLEAR POWER PLANT

## MONTICELLO NUCLEAR GENERATING PLANT

# PALISADES NUCLEAR PLANT

## POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

## PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

### DOCKET NOS. 50-331, 50-305, 50-263, 50-255, 50-266, 50-301, 50-282, AND 50-306

The U.S. Nuclear Regulatory Commission (NRC) staff is reviewing the Nuclear Management Company's (NMC) request for approval of its Quality Assurance Topical Report, Rev 0a, submitted by letter dated October 31, 2003, as supplemented on July 23, 2004. The NRC is reviewing Enclosure 6 of the submittal as a separate task and as such, a request for additional information regarding Enclosure 6 was sent by letter dated June 20, 2004, to which NMC responded on July 23, 2004. We have completed our initial review of the remainder of the submittal and have determined that the following information is needed to continue our review.

1. Assessment

Section C of the proposed quality assurance (QA) topical describes NMC's assessment program. The QA topical defines the term "independent assessment" as being synonymous with the term "audit," as used in the context of Criterion XVIII of Appendix B to 10 CFR Part 50.

American National Standards Institute (ANSI) standard N18.7-1976 (alternately ANS-3.2), "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants," specifies a maximum period of two years to complete an audit cycle covering all safety-related functions. NRC Regulatory Guide (RG) 1.33, "Quality Assurance Program Requirements," Revision 2, amplifies this guidance in specifying minimum audit frequencies of 6 months for audits of corrective actions affecting safetyrelated equipment, 12 months for audits of facility conformance with technical specifications (TS), and 12 months for audits of performance, training, and qualifications of the facility staff. In order for the staff to determine the acceptability of the proposed assessment program, the following additional information is requested concerning the bases for this exception to ANSI N18.7-1976 and NRC RG 1.33, Rev. 2.

- 1.1 Under the proposed performance-based program, there is no minimum period for completing audits of safety-related functions. Describe in further detail, the systemic attributes of the process for scheduling audits. Identify the applicable procedures that define the independent assessment process and provide these procedures for staff review.
- 1.2 Describe the methodology that will be used as the bases for determining the schedule for independent assessments. Describe the quantitative performance metrics that will indicate whether performance of each topic identified in Table 1 of the QA topical is declining and needs to be assessed.
- 1.3 With regard to NRC RG 1.33, provide the bases for concluding that effective oversight of the activities addressed by regulatory position C.4 (i.e., corrective actions, TS conformance, personnel performance, training, and qualifications) will be maintained at a level commensurate with the safety significance of these activities, in lieu of specifying a minimum assessment frequency.
- 1.4 The submittal proposes to eliminate the independent review program, described in section 4.3 of N18.7. If this program were eliminated, as proposed in Enclosure 6 of the submittal, describe the oversight process that would ensure that the performance and effectiveness of the assessment and independent assessment programs are maintained.

#### 2. Qualification and Training of Personnel

Section A.7.3 of the proposed QA topical identifies NMC commitments to nuclear quality standards other than ANSI/ASME NQA-1, "Quality Assurance Requirements for Nuclear Facility Applications." For qualification and training of personnel, NMC commits to follow the guidance of NRC RG 1.8, "Qualification and Training of Personnel for Nuclear Power Plants," Rev. 3, May 2000.

NRC RG 1.8, Revision 3 endorses ANS 3.1-1993, "Selection, Qualification, and Training of Personnel for Nuclear Power Plants," and establishes two regulatory positions with respect to this standard. In committing to NRC RG 1.8, Revision 3, the QA topical takes exception to both regulatory positions and states that NMC does not commit to follow ANS 3.1-1993. The exceptions are not specified in the QA topical, but reference is made to site TSs.

Current QA programs for NMC plants do not commit to follow Revision 3 of NRC RG 1.8. Point Beach commits to NRC RG 1.8, Revision 0 (1971); Monticello and Prairie Island commit to NRC RG 1.8, Revision 1 (1975); Duane Arnold commits to NRC RG 1.8, Revision 1-R (1977); Palisades and Kewaunee commit to NRC RG 1.8 (1980 Draft). It appears that commitments to NRC RG 1.8 are incorporated in both the plant TSs and the quality assurance program. Consequently, changes in commitments to NRC RG 1.8 would be subject to two regulatory change processes.

- 2.1 Confirm plant-specific commitments to NRC RG 1.8 and ANS 3.1 will remain unchanged as the result of the proposed QA program consolidation or, alternately, provide the bases for changing these commitments.
- 2.2 Provide clarification as to the intent of including a commitment to Revision 3 of NRC RG 1.8 in the QA topical, in addition to the commitments made in the TSs.

#### 3. Qualification of Nondestructive Examination Personnel

Section A.5 of the proposed QA topical describes provisions for personnel training and qualification. Exception is taken to NQA-1 Supplementary Requirement 2S-2, which specifies the use of American Society of Nondestructive Testing Recommended Practice SNT-TC-1A for qualification of all personnel who perform examinations covered by that supplement. As stated, the exception allows use of American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code(s) or national standards in lieu of SNT-TC-1A.

Regulatory position C.3 of NRC RG 1.58, "Qualification of Nuclear Power Plant Inspection, Examination, and Testing Personnel," Revision 1, which is followed by most NMC plants, specifies that SNT-TC-1A should be used in conjunction with (i.e., in addition to) the additional provisions of the ASME Code. This position would also apply to other national standards.

- 3.1 Address regulatory position C.3 of NRC RG 1.58 with respect to use of codes other than SNT-TC-1A for qualification of nondestructive examination personnel.
- 4. Hoisting and Rigging

NMC plants currently follow the guidance of ANSI N45.2.2, "Packaging, Shipping, Receiving, Storage, and Handling of Items for Nuclear Power Plants During the Construction Phase," Section 7 for handling safety-related items. Guidance is provided for the handling of items to minimize damage and to preserve the quality of the item and container. ANSI N45.2.2 incorporates, by reference, industry standards applicable to these guidelines, such as the ANSI B30 series related to hoisting and rigging.

By taking exception to NQA-1, Subpart 2.15, "QA Requirements for Hoisting, Rigging, and Transporting of Items for Nuclear Power Plants," NMC effectively withdraws the current commitment to ANSI N45.2.2 for hoisting and rigging. Changes to these commitments would no longer be reviewed by the staff through the regulatory change control process of 10 CFR 50.54(a)(3).

4.1 Provide additional information describing the proposed exception to the guidance of N45.2.2 for material handling. Identify and describe NMC commitments to standards establishing quality assurance requirements for hoisting and rigging, the licensing

documents containing these commitments, and the process for controlling changes to these commitments.

#### 5. <u>Records</u>

NQA-1, Supplementary Requirement 17S-1 classifies records as lifetime (i.e., permanent) records and nonpermanent records. Section B.15 of the proposed QA topical states that NMC uses the list of lifetime records in NQA-1, Appendix 17A-1, supplemented by the recommended retention times established in NRC RG 1.28, "Quality Assurance Program Requirements (Design and Construction)," position C.2 (Table 1).

- 5.1 NRC RG 1.28 does not provide retention times for operation records. Provide retention times for records listed in Appendix 17A-1, but not in NRC RG 1.28.
- 5.2 NMC plants currently follow the guidance of N45.2.9, which provides guidance for retention times for both permanent and nonpermanent records. NQA-1 (Appendix 17A-1) only lists permanent records. Provide the bases for establishing retention times for nonpermanent operations records.
- 5.3 Clarify how 10 CFR 71 and 10 CFR 72 recordkeeping requirements will be satisfied. Specifically, Section A.7.3 of the proposed QA topical states,"NMC complies with position C.2 [of Regulatory Guide 1.28, Revision 3] for record retention times." Section B.15, second paragraph, states "NMC uses the list of records in Non-mandatory Appendix 17A-1, supplemented by the recommended retention times established in Regulatory Guide 1.28, position C.2 (Table 1), to establish the types of records that will be created and retained in support of plant operation." Section B.15, third paragraph, states "In establishing provisions for records, NMC commits to compliance with NQA-1, 1994, Basic Requirement 17 and Supplement 17S-1." Position C.2 and Supplement 17S-1 do not appear to satisfy the requirements of 10 CFR 71.135 and 10 CFR 72.174. For example, 10 CFR 71.135 requires the licensee to retain records "for 3 years beyond the date when the licensee last engages in the activity for which the quality assurance program was developed," and 10 CFR 72.174 requires "records must include closely related data such as qualifications of personnel, procedures, and equipment" and "must be maintained...until the NRC terminates the license or CoC."