

July 30, 2004

Mr. D. L. Koehl
Site Vice President
Nuclear Management Company, LLC
6610 Nuclear Road
Point Beach Nuclear Plant
Two Rivers, WI 54241

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL
APPLICATION (TAC. NOS. MC2099 AND MC2100)

Dear Mr. Koehl:

By letter dated February 25, 2004, Nuclear Management Company, LLC (NMC or the applicant) submitted an application pursuant to 10 CFR Part 54, to renew the operating licenses for Point Beach Nuclear Plant, Units 1 and 2, for review by the U.S. Nuclear Regulatory Commission (NRC). The NRC staff is reviewing the information contained in the license renewal application (LRA) and has identified, in the enclosure, areas where additional information is needed to complete the review. Specifically, the enclosed requests for additional information (RAIs) are from Section 3.2 Engineered Safety Features and Section 3.4 Steam and Power Conversion System.

These RAIs, in a draft format, have been provided to Mr. Jim Knorr of your staff on June 28, 2004. The NRC staff has discussed draft versions of these RAIs, via a telephone conference call, to provide clarifications to the NMC staff on July 13, 2004. Your responses to these RAIs are requested within 30 days from the date of this letter. Mr. Knorr has agreed to this request. If needed, the NRC staff is willing to meet or discuss with NMC again prior to the submittal of the applicant's responses to provide clarifications to the staff's RAIs.

If you have any questions, please contact me at 301-415-2232 or e-mail mjm2@nrc.gov.

Sincerely,

/RA/ Samson Lee for

Michael J. Morgan, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos.: 50-266 and 50-301

Enclosure: As stated

cc w/enclosure: See next page

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Sincerely,
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Michael J. Morgan, Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

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ADAMS Accession No.: ML042150431

Document Name: C:\ORPCheckout\FileNET\ML042150431.wpd

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**REQUEST FOR ADDITIONAL INFORMATION
POINT BEACH NUCLEAR PLANT, UNITS 1 & 2
LICENSE RENEWAL APPLICATION**

Section 3.2 Engineered Safety Features

RAI 3.2-1

In LRA Tables 3.2.2-1, 3.2.2-2, and 3.2.2-3, stainless steel heat exchangers, with heat transfer as the intended function, are identified with only internal environments and associated aging effects. The external environment is listed as not applicable. The applicant is requested to explain why for these components the external environment and associated aging effects need not be identified. The applicant is also requested to address the similar question for the stainless steel heat exchangers in Tables 3.2.2-1 and 3.2.2-2, which have pressure boundary as their intended function.

RAI 3.2-2

In LRA Tables 3.2.2-1 and 3.2.2-2, carbon/low alloy steel tank components are identified with only external environments and the associated aging effects. The internal environment is listed as N/A. On the other hand, stainless steel tank components are identified with internal environments with the associated aging effects. The applicant is requested to explain the material construction of the tank, and the environments to which it is exposed, to support the information provided in the tables.

Section 3.4 Steam and Power Conversion System

RAI 3.4-1

In LRA Table 3.4.2-3, stainless steel heat exchangers, with either heat transfer or pressure boundary as the intended function, are identified with only internal environments and associated aging effects. The external environment is listed as not applicable. The applicant is requested to explain why for these components the external environment and associated aging effects need not be identified.

RAI 3.4-2

In the first quarter of 2003, Point Beach Nuclear Plant entered the Multiple/Repetitive Degraded Cornerstone Column (Column IV) of the Action Matrix of NRC Inspection Manual Chapter 0305, "Operating Reactor Assessment Program," as a result of a high significance (Red) inspection finding involving the potential for a common mode failure of the auxiliary feedwater system (AFW) following a loss of the instrument air system. A second Red inspection finding (Yellow for Unit 1 and Red for Unit 2) was subsequently identified which involved the potential common mode failure of the AFW pumps due to plugging of the recirculation line pressure reduction orifices. In view of the aforementioned inspection findings and the subsequent

Enclosure

corrective actions for the two AFW issues, the applicant is requested to address the following questions in the context of the license renewal application:

- (a) Clarify whether the instrument air system and the recirculation line pressure reduction orifices are in scope for the LRA evaluation, and that appropriate AMPs have been identified to manage the components.
- (b) For aging management evaluation as summarized in LRA Table 3.4.2-3 for the auxiliary feedwater system, discuss any impact on the scoping of mechanical components in the auxiliary feedwater system, and the AMR performed for the affected components, as a result of the recent physical modifications made to resolve the auxiliary feedwater pump room Appendix R issues.

Point Beach Nuclear Plant, Units 1 and 2

cc:

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