

From: Mahesh Chawla
To: Dale.Vincent@nmcco.com
Date: 4/8/04 4:58PM
Subject: RAIs for L-PI-04-005 Dated Jan 7, 2004

With reference to the above relief request, following is the list of questions the NRC staff would like to discuss with you during a teleconference. Please review these and indicate your availability next week.

By letter dated January 7, 2004, the licensee, Nuclear Management Company (NMC), submitted Request for Relief No. 16 from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, for Prairie Island Unit 2. The NRC staff reviewed the information submitted by the licensee, and based on this review, determined the following information is required to complete the evaluation.

1. Regarding Table 1 on Page 10 of the submittal, what was the UT coverage obtained from the previous interval? If, for any weld, the coverage was less than before, please explain.
2. Regarding Table 1 on Page 10 of the submittal, the percent coverage seems to be inconsistent with the description provided on Pages 2-4. Please clarify.
3. For certain welds, including RC Weld W-6/2LSU on Page 3, the licensee stated, "the weld is included in the boundary examined by VT-2 during pressure testing..." Please provide a brief explanation to state what value has been added through the VT-2 examination. For example, if the results indicated leakage integrity of the component, etc.
4. For MS Weld W-36, the photo in the attached is not clear. Please provide a cross section of the weld. Please explain what prevented the UT examination. The license also stated that a VT-1 was performed on the ID surface. Please explain how a VT-1 can be used to detect flaws. Please explain why surface examination was not considered on the ID in lieu of VT-1 to detect flaws.

CC: Jack.Laveille@nmcco.com; Z Fu

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