



A subsidiary of Pinnacle West Capital Corporation

10CFR50.73

Palo Verde Nuclear
Generating Station

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192-01146-DMS/SAB/DJS
July 23, 2004

ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Dear Sirs:

**Subject: Palo Verde Nuclear Generating Station (PVNGS)
Units 1, 2, and 3
Docket No. STN 50-528, 50-529 and 50-530
License No. NPF-41, NPF-51 and NPF-74
Licensee Event Report 1-2004-005-00**

Attached please find Licensee Event Report (LER) 50-528/2004-005-00 that has been prepared and submitted pursuant to 10CFR50.73. This LER reports findings related to missed Surveillance Tests for the Shutdown Cooling Pressure Interlocks for Units 1, 2 and 3.

In accordance with 10CFR50.4, a copy of this LER is being forwarded to the NRC Region IV Office and the Senior Resident Inspector. If you have questions regarding this submittal, please contact Daniel G. Marks, Section Leader, Regulatory Affairs, at (623) 393-6492.

Arizona Public Service Company makes no commitments in this letter. The corrective actions described in this LER are not necessary to maintain compliance with regulations.

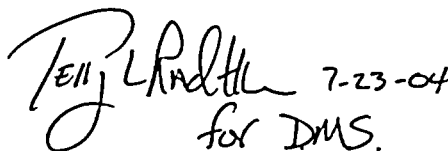
Sincerely,

DMS/SAB/DJS/kg

Attachment

cc: B. S. Mallett
M. B. Fields
N. L. Salgado

NRC Region IV Regional Administrator
NRC NRR Project Manager
NRC Senior Resident Inspector for PVNGS

 7-23-04
for DMS.

IE22

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Palo Verde Nuclear Generating Station Unit 1	2. DOCKET NUMBER 05000528	3. PAGE 1 OF 5
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4. TITLE
MISSED ST ON SHUTDOWN COOLING VALVE RCS PRESSURE INTERLOCKS

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	28	2004	2004	005	00	07	23	2004	PVNGS Unit 2	05000529
									FACILITY NAME	DOCKET NUMBER
									PVNGS Unit 3	05000530

9. OPERATING MODE 1	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)									
	20.2201(b)	20.2203(a)(3)(ii)	50.73(a)(2)(ii)(B)	50.73(a)(2)(ix)(A)						
10. POWER LEVEL 100	20.2201(d)	20.2203(a)(4)	50.73(a)(2)(iii)	50.73(a)(2)(x)						
	20.2203(a)(1)	50.36(c)(1)(i)(A)	50.73(a)(2)(iv)(A)	73.71(a)(4)						
	20.2203(a)(2)(i)	50.36(c)(1)(ii)(A)	50.73(a)(2)(v)(A)	73.71(a)(5)						
	20.2203(a)(2)(ii)	50.36(c)(2)	50.73(a)(2)(v)(B)	OTHER - Voluntary Specify in Abstract below or in NRC Form 366A						
	20.2203(a)(2)(iii)	50.46(a)(3)(ii)	50.73(a)(2)(v)(C)							
	20.2203(a)(2)(iv)	50.73(a)(2)(i)(A)	50.73(a)(2)(v)(D)							
	20.2203(a)(2)(v)	XX 50.73(a)(2)(i)(B)	50.73(a)(2)(vii)							
	20.2203(a)(2)(vi)	50.73(a)(2)(i)(C)	50.73(a)(2)(viii)(A)							
20.2203(a)(3)(i)	50.73(a)(2)(ii)(A)	50.73(a)(2)(viii)(B)								

12. LICENSEE CONTACT FOR THIS LER

NAME Daniel G. Marks, Section Leader, Regulatory Affairs	TELEPHONE NUMBER (Include Area Code) 623-393-6492
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED				15. EXPECTED SUBMISSION DATE		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO						

16. ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On May 28, 2004, during a review of Condition Report number 2687507 it was determined that Palo Verde was not testing SDC isolation valve interlocks as described in UFSAR section 7.6, nor were these safety-related interlocks being subjected to a complete channel calibration/functional test as intended by NRC GL96-01. The Technical Specification surveillance requirement is 3.4.15.2.

Surveillance tests 36ST-9SI04 and 36ST-9SI05 were revised to match the test procedure described in UFSAR section 7.6.2. The tests have been successfully completed in all three units. A review of other non-Plant Protection System safety-related interlocks, inhibits, permissives or bypass circuits that may not have been included in surveillance tests will be conducted.

Previous similar events were reported in LERs 3-2003-003, 2-2003-003 and 1-2001-005.

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Palo Verde Nuclear Generating Station Units 1, 2, and 3	05000528	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 5
	05000529 05000530	2004	-- 005	-- 00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

1. REPORTING REQUIREMENT(S):

This LER 528/2004-005-00 is being submitted to report a condition which was prohibited by the plant's Technical Specifications. This report applies to Units 1, 2, and 3.

Specifically, the tests conducted by Technical Specification Surveillance Requirements 3.4.15.2 to verify the Shut Down Cooling (SDC) isolation valve interlocks were inadequate.

2. DESCRIPTION OF STRUCTURE(S), SYSTEM(S) AND COMPONENT(S):

Shutdown Cooling System (EIS: BP) suction line valve interlocks are designed to act as permissives. The Shutdown Cooling System Suction Line Valve Interlocks permit the isolation valves to be opened below a certain RCS pressure. The Shutdown Cooling System Suction Line Valve Interlocks receive power from the emergency busses should there be a loss of all AC power. This is to assure that the interlocks and valves will be able to operate under all operating conditions. The SCS is a low temperature, low pressure system used to remove decay heat from the RCS. Cooldown of the RCS is accomplished via the steam generator down to about 350°F and about 400 psia. Below these values the SCS is used to cool the RCS to refueling temperatures and to maintain these conditions for extended periods of time. To preclude over-pressurization, there are redundant, motor driven, interlocked, isolation valves on each suction line. The interlocks prevent the suction line isolation valves from being opened if RCS pressure has not decreased below 410 psia. These interlocks are redundant so that any single failure will not cause a suction line and heat exchanger to be subjected to pressures greater than design pressure. The interlock cannot be overridden so that operator action cannot inadvertently subject the SCS to RCS pressure. In addition, no single failure can prevent the operator from aligning the valves, on at least one suction line, for shutdown cooling after RCS pressure requirements are satisfied. Redundant relief valves are provided on the suction lines to prevent or mitigate over-pressurization from pressure transients. These transients can be caused by inadvertent starting of HPSI pumps, charging pumps, inadvertent energizing of pressurizer backup heaters, or a combination

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	05000530				

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of these. The relief valves are set at 467 psig to ensure the system stays below its design limits. The SDC valve interlocks are safety-related and designed to protect the SDC system from over-pressurization.

3. INITIAL PLANT CONDITIONS:

On May 28, 2004, Palo Verde Units 1, 2 and 3 were in Mode 1 (POWER OPERATION), operating at approximately 100 percent power. There were no major structures, systems, or components that were inoperable at the start of the event that contributed to the event. There were no failures that rendered a train of a safety system inoperable and no failures of components with multiple functions were involved.

4. EVENT DESCRIPTION:

On May 28, 2004, during a review of Condition Report number 2687507 it was determined that Palo Verde was not testing SDC isolation valve interlocks as described in UFSAR section 7.6, nor were these safety-related interlocks being subjected to a complete channel calibration/functional test as intended by NRC GL96-01. The Technical Specification surveillance requirement is 3.4.15.2.

5. ASSESSMENT OF SAFETY CONSEQUENCES:

S.R. 3.4.15.2 verifies the SDC system open permissive interlock prevents the valves from being opened with a simulated or actual RCS pressure signal > 410 psia every 18 months.

The open permissive interlocks are installed on the three SDC Suction piping isolation valves on each train; Train A SIAUV651 (upstream), SICUV653 (middle), SIAUV655 (downstream), Train B SIBUV652 (upstream), SIDUV654 (middle), SIBUV656 (downstream). The functional intent of the open permissive interlock (410 psia setpoint) is to ensure the RCS pressure will not pressurize the SDC system beyond 125% of its design pressure of 485psig, and ensure the SDC relief valves will not lift.

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Per Site Procedure "Cold Shutdown to Hot Standby Mode 5 to Mode 3", RCS pressure is maintained less than 410 psia until the Low Temperature Over Pressurization (LTOP) relief valves are isolated and the power supply disconnects for SICUV653 and SIDUV654 are Open and Locked per 40AC-0ZZ06 "Locked Valve, Breaker, and Component Control".

Removal of power from these valves also removes control power and has the effect of precluding inadvertently opening the valves from either spurious failure or inadvertent hand-switch operation. Down powering the valve is functionally equivalent to the open permissive interlock. If a downstream isolation valve's interlock failed to ensure the valve would not open, and the valve was inadvertently opened from the control room or remote shutdown panel, the SDC system would remain isolated from RCS pressure by the remaining isolation valves.

If an upstream isolation valve's interlock failed to ensure the valve would not open, and the valve was inadvertently opened from the control room or remote shutdown panel, the SDC suction line would be pressurized up to the middle isolation valve. This section of piping is rated for RCS pressure, and the thermal relief valve's setpoint is 2485 + 75 psig. Thus no LOCA initiation or loss of SI system function would occur. No impacts were identified, assuming a failure of the open permissive interlock, that would result in an over-pressurization of the SDC system beyond design limits.

Based on the analysis, the use of the Surveillance test that was not in full compliance with the testing procedure described in the UFSAR would not have prevented the fulfillment of the safety function and would not result in a safety system functional failure as defined by 10CFR50.73(a)(2)(v).

6. CAUSE OF THE EVENT:

The cause of the event was human error. In the process of revising the Surveillance Test, the author used faulty logic in responding to questions in the 10 CFR 50.59 Screening process. The author and the reviewer both failed to recognize that SR 3.4.15.2 and its Basis state that the SDC open permissive interlock must be verified to prevent the valves from being opened with a simulated or actual RCS pressure signal of >410 PSIA every 18 months. UFSAR section 7.6.2.1.1 includes a specific test

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procedure that is intended to be used to comply with these requirements. Surveillance Procedures SDC Interlocks Loop & Alarm Calibration (36ST-9SI04 and 36ST-9SI05) were changed in 1995 to delete valve interlock logic verification and only checked alarm contact status.

No unusual characteristics of the work location (e.g., noise, heat, poor lighting) directly contributed to this event. No procedural errors contributed to this event.

7. CORRECTIVE ACTIONS:

Surveillance tests 36ST-9SI04 and 36ST-9SI05 were revised to match the test procedure described in UFSAR section 7.6.2. The tests have been successfully completed in all three units.

A review of other non-Plant Protection System safety-related interlocks, inhibits, permissives or bypass circuits that may not have been included in surveillance tests will be conducted.

8. PREVIOUS SIMILAR EVENTS:

In the past three years, previous similar events were reported in LERs 3-2003-003, 2-2003-003 and 1-2001-005.