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10 CFR 50.73

July 22, 2004 2130-04-20144

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555 - 0001

> Oyster Creek Generating Station Facility Operating License No. DPR-16 NRC Docket No. 50-219

Subject: Licensee Event Report 2004-003-00: Actuation of Reactor Protection System due to Spurious HI-HI Trip Signals on Intermediate Range Monitors Caused by Electromagnetic Interference

Enclosed is Licensee Event Report 2004-003, Revision 0. This event did not affect the health and safety of the public or plant personnel.

If any further information or assistance is needed, please contact David Fawcett at 609-971-4284.

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C. N. Swenson¹ Vice President, Oyster Creek Generating Station

CNS/DIF Enclosure

cc: H. J. Miller, Administrator, USNRC Region I
 P. S. Tam, USNRC Senior Project Manager, Oyster Creek
 R. J. Summers, USNRC Senior Resident Inspector, Oyster Creek
 File No. 04106

NRC FORM 366 (7-2001) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)						APPROVED BY OMB NO. 3150-0104 EXPIRES 7-31-2004 Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by intermet e-mail to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB- 10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.								
														
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X YES (If yes	s, comple	ete EXPr	ECTED S	UBMISSION	DAT	E)		0		DATE		09	30	2004
approxima channels the IRM cl down as d The safety Technical operable.	7, 2004, ately 2% 13, 14, a hannels designed y signific Specific Off-site	, at 00: 6 power and 18. 5 due to d. cance of cation li e power	31 hour r was ca . The s loose c of this e limits was r was av	rs, with the aused by a spurious ac connection event is cor vere mainta vailable. C	e Rea a spur ctuations, da nside ained. Opera	actor Mod irious act ion was o amaged ered mini . There ator perfe	de sw tuatio cause cablir imal. was r orma	vitch in t on of Inte ed by ele ng, and The pla no radio ince was	he arm ectr def ant i actions sa	Startup pos rediate Ran romagnetic iciencies in responded a ive release.	ge Neu interfer grounc as desi . All sa	itron M rence. ding. T igned fo ifety sy	or scram fro lonitor (IRM) The EMI aff The reactor s or this type o stems were placed.	ected hut of event.

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NRC FORM 366 (7-2001)

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IC FORM 366/	5A U.S. NUCLEAR REGULATORY COMMISSIO						
E Sy -	LICENSE	E EVENT RE	EPORT (L	.ER)			
	1. FACILITY NAME	2. DOCKET	e e	6. LER NUMBER	T	3. PAGE	
Oyster (Creek, Unit 1	05000219	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
	,		2004	- 003 -	00	2 OF 3	
	·····			···		L	
	E (If more space is required, use additional copies	of NRC Form 366A	<i>v</i>				
Descriptic	on of Event						
	27, 2004, at 00:31 hours, with the Reacto						
	1), and IRM channel 18 (RPS Channel 2) cram. The reactor was at approximately						
	ance outage. The SRMs were being drive						
	tor scram shut down the reactor as desig						
	I control rods inserted to 00. Level control e transient. Pressure control was stable.						
•		Ομειαίοι αυτισ	113 WOLC	duuluanuu	աթառբ	100000103.	
Analysis o	of Event						
	ty significance of this event is considered						
	 and responded as designed. Technical nor any effect on the health and safety of 						
			Similar L.			,.	
Cause of							
	n of the Reactor Protection System was con systems. The Hi-Hi/INOP signals on the time of the signal on the time of the signal of the time of time of the time of time o						
interferen	nce) induced spike. The EMI spiking on t	the IRM instrum	nentation was	as apparently c	caused by	loose connectors,	
cable dan	mage, and/or high resistance in IRM grou	und path. A Roc	ot Cause Ev	aluation is in r	progress.		
Corrective	e Actions:			·			
Interim							
1.	The SRM and IRM signal cable conne	ectors were insr	pected and t	tightened as n	ecessary.		
2.	The damaged cable sections were removed or repaired.						
3.	One IRM pre-amplifier, with a high res	sistance to grou	ind connecti	ion, was repla	ced.		
Long Terr	m						
A Ro	ot Cause Evaluation is in progress.						
	· -						

LICENSEE EVENT REPORT (LER)								
_	1. FACILITY NAME	2. DOCKET	6. LER NUMBER 3. PAGE					
0	Dyster Creek, Unit 1	05000219	YEAR SEQUENTIAL REVISION NUMBER NUMBER					
			2004 - 003 - 00 3 OF 3					
	RATIVE (If more space is required, use add ditional Information	itional copies of NRC Form 366A)	· · · ·					
nuu								
A.	Failed Components:							
	None							
В.	Previous similar events:							
	LER 92-007-00, Reactor Scram Ca	aused by Electrical Noise fr	om a Failed IRM Bypass Switch During Plant Startur					
	There have been additional cases reactor was critical. An IRM spikin cause evaluation is in progress.	of IRM spiking problems, b g trend has been identified	ut they did not result in a reactor scram while the through the Corrective Action Program. A root					
C.	Identification of components referr	ed to in this Licensee Even	t Report:					
	Components	IEEE 805 System I	IEEE 803A Function					
	Neutron Monitors	IG	RI					

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