

**Virginia Electric and Power Company  
North Anna Power Station  
P. O. Box 402  
Mineral, Virginia 23117**

July 19, 2004

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555-0001

Serial No.: 04-355  
NAPS: MPW  
Docket No.: 50-339  
License No.: NPF-7

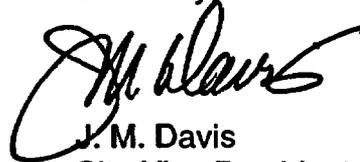
Dear Sirs:

Pursuant to 10CFR50.73, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Power Station Unit 2.

Report No. 50-339/2004-002-00

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Sincerely



J. M. Davis  
Site Vice President  
North Anna Power Station

Enclosure

Commitments contained in this letter: None

cc: United States Nuclear Regulatory Commission  
Region II  
Sam Nunn Atlanta Federal Center  
61 Forsyth Street, SW, Suite 23 T85  
Atlanta, Georgia 30303-8931

Mr. M. T. Widmann  
NRC Senior Resident Inspector  
North Anna Power Station

*IF22*

**LICENSEE EVENT REPORT (LER)**

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1) <b>NORTH ANNA POWER STATION , UNIT 2</b>		DOCKET NUMBER (2) <b>05000 - 339</b>	PAGE (3) <b>1 OF 3</b>
---	--	---	---------------------------

TITLE (4)  
**MANUAL REACTOR TRIP DUE TO CONTROL BANK "D" GROUP STEP COUNTER INOPERABLE**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCUMENT NUMBER
05	29	2004	2004	-- 002 --	00	07	19	2004	FACILITY NAME	DOCUMENT NUMBER
										05000-
										05000-

<b>OPERATING MODE (9)</b> 3  <b>POWER LEVEL (10)</b> 0 %	<b>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply) (11)</b>									
		20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)		
		20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)		50.73(a)(2)(x)		
		20.2203(a)(1)		50.36(c)(1)(i)(A)	X	50.73(a)(2)(iv)(A)		73.71(a)(4)		
		20.2203(a)(2)(i)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)		73.71(a)(5)		
		20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)		OTHER		
		20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)		Specify in Abstract below or in NRC Form 366A		
		20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)				
		20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)				
	20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)					
	20.2203(a)(3)(i)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)					

LICENSEE CONTACT FOR THIS LER (12)

NAME <b>J. M. Davis, Site Vice President</b>	TELEPHONE NUMBER (Include Area Code) <b>(540) 894-2101</b>
---	---

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
A	AA	CTR	W120	Y					

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO					

**ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)**

On May 29, 2004, at 0210 hours, a manual reactor trip was initiated as a result of having Control Bank "D" Group demand position indicators inoperable. A mismatch of the Control Bank "D" Group 1 and 2 demand position indicators greater than two steps occurred. This was the result of the Control Bank "D" Group 2 demand position indicator cover not being in its proper position (i.e., snapped down) such that the counter mechanical cams were not engaged. At 0955 hours, an 8-hour Non-Emergency Report was made to the NRC in accordance with 10 CFR 50.72 (b)(3)(iv)(A). This event is reportable pursuant to 10 CFR 50.73 (a)(2)(iv)(A) for a condition that resulted in an manual actuation of the reactor protection system. This event posed no significant safety implications because the reactor was subcritical when the reactor trip breakers were opened. Therefore, the health and safety of the public were not affected by this event.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

FACILITY NAME (1)  NORTH ANNA POWER STATION UNIT 2	DOCKET  05000 - 339	LER NUMBER (6)			PAGE (3)  2 OF 3
		YEAR  2004	SEQUENTIAL NUMBER  --002 --	REVISION NUMBER  00	

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

**1.0 DESCRIPTION OF THE EVENT**

On May 29, 2004, Unit 2 was in Mode 3, zero percent power, preparing for restart following a refueling outage. Unit 2 reactor coolant temperature and pressure were 540 degrees Fahrenheit and 2246 psig, respectively. As part of the performance for Rod Drop Time Measurement the Control Bank "D" Groups 1 and 2 control rods were being withdrawn from the core. At 0202 hours, the Group Step Counters were declared inoperable as a result of Control Bank "D" Group 1 demand position indicator (EIS System AA, Component CTR) reading four steps while Control Bank "D" Group 2 demand position indicator was reading one step. At 0210 hours, the reactor trip breakers (RTB) were manually opened in accordance with Technical Requirement 3.1.3 due to a mismatch of the Control Bank "D" Group demand position indicators greater than two steps.

The demand step indicators were inspected for Control Bank "D" Groups 1 and 2 with no problems noted. The RTBs (EIS BKR) were subsequently tested satisfactorily and closed. The Control Bank "D" Groups 1 and 2 controls were withdrawn two steps, inserted two steps and then positioned back to zero with no problems noted. At 0350 hours, the rod drop time measurements resumed. Rod drop time measurements for the "A" and "C" Control Bank Groups continued and were completed satisfactorily. It was determined the problem with the Control Bank "D" Group 2 demand step counter was the result of the cover not being in its proper position. The cover was snapped closed to its proper position. The RTBs were tested satisfactorily and the rod drop time measurement for Control Bank "D" Groups 1 and 2 control rods resumed. At 1537 hours, rod drop time measurements were completed satisfactorily.

Following repairs and testing of station equipment the unit restart commenced. Unit 2 was placed on-line at 1112 hours on May 30, 2004.

**2.0 SIGNIFICANT SAFETY CONSEQUENCES AND IMPLICATIONS**

This event posed no significant safety implications because the reactor was subcritical when the RTBs were opened. Compliance with all Technical Requirements was achieved. Therefore, the health and safety of the public were not affected by this event.

An 8-hour non-emergency report was made to the NRC at 0955 hours in accordance with 10 CFR 50.72 (b)(3)(iv)(A). This event is reportable pursuant to 10 CFR 50.73 (a)(2)(iv)(A) for a condition that resulted in a manual actuation of any engineered safety feature including the reactor protection system.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

FACILITY NAME (1)  NORTH ANNA POWER STATION UNIT 2	DOCKET  05000 - 339	LER NUMBER (6)			PAGE (3)  3 OF 3
		YEAR  2004	SEQUENTIAL NUMBER  --002 --	REVISION NUMBER  00	

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

**3.0 CAUSE**

The manual reactor trip was the result of having a mismatch in Group Step Demand Counters. This was the result of the Control Bank "D" Group 2 demand position indicator cover not being in its proper position (i.e., snapped down) such that the counter mechanical cams were not engaged. This position prevented the demand step counter from operating properly. The improper condition of the demand position indicator cover is attributed to worker practices following a previous adjustment.

**4.0 IMMEDIATE CORRECTIVE ACTION(S)**

In each instance the Control Room Operators performed the required actions of the Technical Requirements within the specified times. Troubleshooting of the Control Bank "D" Group 1 and 2 demand position indicators was performed. The Control Bank "D" Group 2 demand position indicator cover was placed in the correct position.

**5.0 ADDITIONAL CORRECTIVE ACTIONS**

A Training Information Bulletin has been developed to heighten awareness regarding the need to snap the demand step counter covers closed. This event will be covered in Continuing Training classes for Operations, Instrument & Controls, and Reactor Engineering. Operations and testing procedures will be enhanced by adding a caution concerning the proper position of the demand step counter cover.

**6.0 ACTIONS TO PREVENT RECURRENCE**

The actions stated above are sufficient to preclude recurrence.

**7.0 SIMILAR EVENTS**

The following events were reported via LERs however, the cause for these events was equipment failure versus an improper position of the demand step counter cover.

LER N2-98-001-00 dated 04/30/98 and LER N2-01-002-00 dated 04/24/01.

**8.0 ADDITIONAL INFORMATION**

At the time of this event Unit 1 was operating at 100 percent power and was not affected by this event.