

Duke Energy Company Oconee 1, 2, 3
Entergy Operations, Inc. ANO-1
Florida Power Corporation Crystal River 3



AmerGen Energy Company, LLC
FirstEnergy Nuclear Operating Company
Framatome ANP

TMI-1
D-B

Working Together to Economically Provide Reliable and Safe Electrical Power

July 15, 2004
NRC:04:034
OG:04:1849

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Schedule for New B&WOG Topical Report to Address Steam Generator Tube Integrity for a Selected Large Break LOCA.

- Ref.: 1. Letter, William R. McCollum (B&WOG), to Document Control Desk (NRC), "Review of BAW-2374," OG:03:1833/NRC:03:014, March 13, 2003.
- Ref.: 2. Letter, Herbert N. Berkow (NRC), to James F. Mallay (Framatome ANP), "Review of Topical Report BAW-2374, 'Risk-Informed Assessment of Once-Through Steam Generator Tube Thermal Loads Due to Breaks in Reactor Coolant System Upper Hot Leg Large Bore Piping'," (TAC No. MC8187) May 15, 2003.

On March 13, 2003, the B&WOG withdrew from NRC review the topical report BAW-2374, Revision 1, "Risk-Informed Assessment of Once-through Steam Generator Tube Thermal Loads due to Breaks in Reactor Coolant System Upper Hot Leg Large Bore Piping." (See Reference 1.) The B&WOG currently plans to submit a replacement topical report by December 30, 2005. A listing of the tasks involved in the development of the topical report is enclosed.

The new topical report will include the risk-related material from BAW-2374, Revision 1. In addition, it will include the results of a deterministic analysis to demonstrate compliance with 10CFR50.46, including the criterion on long term cooling. The enclosed work plan is based on detailed discussions with the NRC held in April 2004. This new approach differs markedly from the agreement reached with the NRC in a meeting with the B&WOG on February 6, 2003 and essentially confirmed in a May 15, 2003 letter from the NRC. (See Reference 2.) Because of these differences and the magnitude of this revised approach, the B&WOG will be seeking further agreement with the NRC on the scope of the effort and the schedule on which this work can be completed.

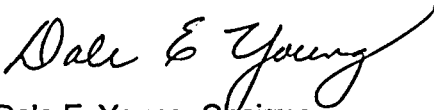
Framatome ANP B&W Owners Group
3315 Old Forest Road
Lynchburg, VA 24501
Phone: 434-832-2981 Fax: 434-832-2475

D045

The B&WOG proposes to meet with the NRC in mid-October to discuss our progress on several key analyses, including tube failure, cladding integrity, and initialization of the LOCA evaluation. This meeting would focus on the enclosed work plan.

We look forward to working with the NRC so this matter is resolved to the satisfaction of all parties involved.

Very truly yours,

A handwritten signature in cursive script that reads "Dale E. Young".

Dale E. Young, Chairman
B&WOG Executive Committee

Enclosure

cc: D. G. Holland
Analysis Committee
Steering Committee
Project 693

Activity ID	Activity Description		
		2004	2005
Major Project Milestones			
1000	Project Start	◆ Project Start	
1100	NRC Meeting	◆ NRC Meeting	
1200	Analysis Committe Meeting	▲ Analysis Committe Meeting	
1300	Analysis / SG / Steering Buy-in	◆ Analysis / SG / Steering Buy-in	
1400	Project Schedule to NRC	◆ Project Schedule to NRC	
1500	NRC Meeting	◆ NRC Meeting	
1600	Thermal Hydraulic Case Defined	◆ Thermal Hydraulic Case Defined	
1700	NRC Meeting	◆ NRC Meeting	
1720	Issue Draft Report	◆ Issue Draft Report	
1740	Issue Final BAW	Issue Final BAW◆	
Long Term Cooling			
1800	Preliminary Thermal Hydraulics Spreadsheet	▲ Preliminary Thermal Hydraulics Spreadsheet	
2000	Monte Carlo Utility Inputs	▲ Monte Carlo Utility Inputs	
1900	Monte Carlo Evaluation thermal hydraulics	▲ Monte Carlo Evaluation thermal hydraulics inputs	
2200	Determine # of Tubes Ruptured vs. Delta T	▲ Determine # of Tubes Ruptured vs. Delta T	
2100	QA Thermal Hydraulics Spreadsheet	▲ QA Thermal Hydraulics Spreadsheet	
2300	Application of Thermal Hydraulics	▲ Application of Thermal Hydraulics Spreadsheet	
2400	RELAP 5 Transient Analysis	▲ RELAP 5 Transient Analysis	
Doses Evaluation			
2500	Clad Rupture Study	▲ Clad Rupture Study	
2600	Plant Specific Dose Evaluation	Plant Specific Dose Evaluation▲	
Secondary Pipe Integrity			
2700	Plant Specific Waterhammer	▲ Plant Specific Waterhammer	
Prepare Topical Report Text			
2800	Waterhammer Text	▲ Waterhammer Text	
2900	Write Long Term Cooling Text	Write Long Term Cooling Text▲	
3000	Write Dose Text	▲ Write Dose Text	
3100	Committee Review	Committee Review▲ col	

Start Date 01APR04
 Finish Date 20DEC05
 Data Date 01APR04
 Run Date 15JUL04 11:30

▲ Early Bar
 ▲ Progress Bar
 ▲ Critical Activity

BAW1

AREVA

Sheet 1 of 1

Revision B&W-2374
 Topical Report
 All Activities

Date	Revision	Checked	Approved