

July 20, 2004

Mr. Michael R. Kansler, President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING RELIEF
REQUEST NOS. RR-62 AND 3-32 CONCERNING EMBEDDED FLAW REPAIR
METHODS, INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3
(TAC NOS. MC3281 AND MC3282)

Dear Mr. Kansler:

By letter dated May 25, 2004, Entergy Nuclear Operations, Inc. (Entergy) submitted relief requests (RRs) 62, Revision 1, and 3-32, Revision 1, regarding proposed alternative repair methods and examination techniques for Indian Point Nuclear Generating Unit Nos. 2 and 3. Specifically, Entergy has proposed an alternative to the requirements of American Society of Mechanical Engineers Boiler and Pressure Vessel Code to use the embedded flaw process for the repair of reactor vessel head penetrations, if necessary.

The Nuclear Regulatory Commission staff is reviewing your request and has determined that additional information is necessary in order to complete its evaluation. The specific question is in the enclosed request for additional information (RAI). During a telephone call on July 12, 2004, the Entergy staff indicated that a response to the RAI would be provided within 60 days.

Sincerely,

/RA/

Patrick D. Milano, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

Enclosure: As stated

cc w/encl: See next page

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Indian Point Nuclear Generating Unit No. 2 and 3

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Indian Point Nuclear Generating Unit No. 2 and 3

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Indian Point Nuclear Generating Unit No. 2 and 3

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REQUEST FOR ADDITIONAL INFORMATION (RAI)
REGARDING RELIEF REQUESTS 62, REVISION 1, AND 3-32, REVISION 1
ON ALTERNATE REPAIR AND EXAMINATION METHODS
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3 (IP2 AND 3)
DOCKET NOS. 50-247 AND 50-286

Relief Request (RR) 3-32, Revision 1, states in "Proposed Alternative" that: "The embedded flaw repair overlay welds on the inside diameter (ID) and the outside diameter (OD) of the penetration tube material will consist of a minimum of 2 deposited layers of weld, consistent with References 3, 4, and 5 to minimize welding induced residual stresses and material distortion." Reference 4 is the NRC safety evaluation for Palo Verde Nuclear Generating Station, and Reference 5 is not identified. As stated in your May 25, 2004, letter to the NRC, RR 3-32, Revision 1, is similar to RR 62, Revision 1, for IP2. RR 62 makes reference, in part, to the Westinghouse Topical Report, WCAP-15987-P-A, Revision 2. It is not clear how References 4 and 5 apply to RR 3-32. Provide the specific aspects of References 4 and 5 that apply to this RR.

Enclosure