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July 14, 2004

Mr. Robert L. Clark Office of Nuclear Regulatory Regulation U.S. Nuclear Regulatory Commission **ATTN: Document Control Desk** Washington, D.C. 20555-0001

Subject: **Response to Request for Additional Information** Regarding R.E. Ginna Nuclear Power Plant Relief Requests VR-1, VR-2, and VR-13 for The Fourth 10-Year Interval Inspection Docket No. 50-244

- References: (1) Letter from R. Clark, NRC, to R. Mecredy, RG&E, Subject: Request for Additional Information Regarding R.E. Ginna Nuclear Power Plant Relief Requests VR-1, VR-2, and VR-13 for the Fourth 10-year Interval Inspection (Tac No. MB2393), dated June 15, 2004.
 - (2) Letter from R. Mecredy, RG&E, to R. Clark, NRC, Subject: Submittal of Relief Requests VR-1, VR-2, and VR-13 Related to the Requirements of 10CFR50.55a(f), "Inservice Testing Requirements", dated March 18, 2004.

Dear Mr. Clark:

In Reference 1, the NRC provided a Request for Additional Information (RAI) related to proposed relief requests for Ginna Station concerning the ASME Section XI Inservice Testing Program (Reference 2). The purpose of this letter is to provide the response to the questions documented in Reference 1 (see enclosure).

No new commitments are being made in this letter.

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attachments

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Enclosure 1

Response to NRC Request for Additional Information (RAI) Dated June 15, 2004

The response to the RAI will be structured as follows. The items in **bold** below are the questions provided by the NRC in the RAI dated June 15, 2004. A response to each item is then provided by R.E. Ginna.

Relief Request No. VR-1, Revision 1

<u>RAI 1</u>: The licensee states under Relief Request VR-1 "Basis for Relief" Item No. 1, "The design of the system is such that either emergency diesel generators can be isolated and the check valve disassembled with the plant online." Whereas in previously approved Relief Request VR-1 dated June 13, 2000, the licensee stated under "Basis for Relief," "During any mode of plant operation there is no practical means to exercise these valves. The valve closure cannot be verified due to system design. To perform a valve closure verification would require disassembly of mechanical joints in the piping, which would place the diesel in an inoperable condition." The licensee's statements for the same valves under Relief Request VR-1, Revision 0 and VR-1, Revision 1 appears to be contradicting. (a) Please explain, why the testing that was impractical previously can now be performed. (b) Also, if online testing of check valves 5960A/B is practical, then provide the reasoning for not testing these valves at least once every three months as required by the OM-10, para 4.3.2.1.

<u>Response:</u> Check valves 5960 A and 5960 B were not capable of being flow tested (i.e. can not be verified full open or partially open, or full closed) when VR-1, revision 0 was submitted, and remain incapable of being flow tested (exercised full open and full closed) due to system design. Revision 1 to VR-1 is intended to provide the option of performing the disassembly activity endorsed by Generic Letter 89-04, Position 2, "Alternative to Full Flow Testing of Check Valves " during on-line system maintenance, when the diesel generator is already inoperable, in lieu of only during a refueling outage as stated in the current relief request, as well as in OMa-1988, Part 10, paragraph 4.3.2.4, (c).

Since flow testing of check valves 5960 A and 5960 B is not practical, and since disassembly is the only feasible means to demonstrate the necessary obturator movement (i.e. manually exercise full open and full closed), quarterly flow testing of the check valves is not possible, and quarterly disassembly would be undesirable since it would increase diesel generator unavailability.

<u>RAI 2:</u> Please identify what specific paragraph of the OM-10 Standard you are requesting relief (e.g. paragraph 4.3.2.4(c) etc.)

<u>Response:</u> Relief is requested from paragraph 4.3.2.4(c). A new revised Relief Request VR-1 is attached.

RAI 3: Please identify and provide the sizes of the check values for which relief is requested.

Response: Check valves 5960 A and 5960 B are nominally 1.5 inch.

<u>RAI 4:</u> Inservice testing of Emergency Diesel Generator Fuel Oil check valves 5960A and 5960B on a rotating basis or grouping may be performed when both valves are the same size, manufacturer, model number, and material of construction. Please provide the information necessary to verify that both valves 5960A and 5960B are the same size, manufacturer, model number, and material of construction.

<u>Response:</u> Ginna currently has NRC approval to inspect these valves on a sampling basis (the original approval of sampling was documented in an SER dated April 15, 1991). This is based on the fact that the valves are the same size, manufacturer, model number and material of construction (i.e., they are 1.5 inch Jenkins Brothers Model 250 brass/bronze lift check valves).

<u>RAI 5:</u> The Relief Request does not address the safety and risk significance of on-line IST of the check valves. Please address (either in a qualitative or quantitative manner) the potential risk of disassembly and inspection of the check valves on-line compared to the risk when the plant is shutdown.

<u>Response:</u> There is no increase in risk due to the fact that the disassembly and inspection of the valve(s) will be performed during a scheduled maintenance outage of the diesel generator, when it is out of service.

The existing Ginna Station Technical Specification for the diesel generators, LCO 3.8.1, provides for a 7 day allowed outage time. The check valve disassembly can easily be performed within this time window. Also, the inspection and disassembly activities are required to be performed within consideration of the rules involved per 10 CFR 50.65(a)(4).

<u>RAI 6:</u> Provide sufficient information for NRC staff to reach a safety or risk determination with regards to isolation of these check valves when testing on-line. Also provide copies of drawings (P&IDs) showing check valves and isolation valves and provide details how isolation of these check valves will be established.

<u>Response:</u> As noted in response 1 above, these valves will not be flow tested. As shown on the attached drawings (33013-1239 sheet 1 of 2 coordinates C-1 and 33013-1239 sheet 2 of 2 coordinates C-11), there is no difference in isolation of the affected line regardless of the plant status. These valves are in the overflow line from the respective diesel generator fuel oil day tank to the underground main storage tank. There are no isolation valves in the flowpath. Instead, the diesel generator and fuel oil transfer pump will be isolated at the time that the valve inspection and disassembly take place. The line is normally drained.

<u>RAI 7:</u> Provide the leak testing experience and leak tightness reliability of the associated isolation valves and the potential consequences of a loss of isolation capability during disassembly, inspection, and manual exercising of check valves 5960A and 5960B when the plant is on line.

<u>Response:</u> As described above, there are no isolation valves involved, and therefore no consequences.

<u>RAI 8:</u> Based on the risk significance discussed in RAI 5 above, discuss what preventive or compensatory measures are necessary to maintain safety and minimize risk while performing on-line IST.

<u>Response:</u> Since there is no increased risk, as the activity will occur during a scheduled diesel generator minor maintenance outage, the normal risk management tools for work on a diesel generator on-line will be used.

Relief Request No. VR-2

RAI 1: The licensee states under Relief Request VR-2 "Basis for Relief" Item No. 1, "The design of the system is such that either standby auxiliary feedwater pump can be isolated and check valve disassembled with the plant online." Whereas in previously approved Relief Request VR-2 dated June 13, 2000, the licensee stated under "Basis for Relief," "Full-stroke exercising cannot be accomplished during power operation or cold shutdown as this would introduce service water to standby auxiliary feedwater system. Service water does not meet purity requirements for the system or steam generators. Service water would be supplied to steam generators during the required quarterly pump tests if exercising valves 9627A/B was performed." The licensee's statements for same valves under Relief Request VR-2, Revision 0 and VR-2, Revision 1 appears to be contradicting. (a) Please explain, why the testing that was impractical previously can now be performed. (b) Also, if the online testing of check valves 9627A/B is practical, then provide the reasoning for not testing these valves at least once every three months as required by the OM-10, para 4.3.2.1.

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<u>Response:</u> Check valves 9627 A and 9627 B were not capable of being fully flow tested (i.e. can not be verified full open, or full closed, though they can be can be partially open tested) when VR-2, revision 0 was submitted, and remain incapable of being fully flow tested (exercised full open or full closed) due to system design. Revision 1 to VR-2 is merely intended to accomplish the disassembly activity endorsed by Generic Letter 89-04, Position 2, "Alternative to Full Flow Testing of Check Valves" during on-line system maintenance, when the standby auxiliary feedwater train is already inoperable, in lieu of only during a refueling outage as stated in the current relief request, as well as in OMa-1988, Part 10, paragraph 4.3.2.4, (c).

Since full flow testing of check valves 9627 A and 9627 B is not practical, and since disassembly is the only feasible means to demonstrate the necessary obturator movement (i.e. manually exercise full open and full closed), quarterly full flow testing of the check valves is not possible, and quarterly disassembly would be undesirable since it would increase standby auxiliary feedwater system unavailability.

<u>RAI 2:</u> Please identify what specific paragraph of the OM-10 Standard you are requesting relief (e.g. paragraph 4.3.2.4(c) etc.)

<u>Response:</u> Relief is requested from paragraph 4.3.2.4(c). A new revised Relief Request VR-2 is attached.

<u>RAI 3:</u> Please identify and provide the sizes of the check valves for which relief is requested.

Response: Check valves 9627 A and 9627 B are nominally 4 inch.

<u>RAI 4:</u> Inservice testing of Standby Auxiliary Feedwater check values 9627A and 9627B on a rotating basis or grouping may be performed when both values are the same size, manufacturer, model number and material of construction. Please provide the information necessary to verify that both values 9627A and 9627B are the same size, manufacturer, model number and material of construction.

<u>Response:</u> Ginna currently has NRC approval to inspect these valves on a sampling basis (the original approval of sampling was documented in an SER dated April 15, 1991). This is based on the fact that the valves are the same size, manufacturer, model number and material of construction (i.e., they are 4 inch Borg-Warner Model 73490 carbon steel swing check valves).

<u>RAI 5:</u> The Relief Request does not address the safety and risk significance of on-line IST of the check valves. Please address (either in a qualitative or quantitative manner) the potential risk of disassembly and inspection of these check valves on-line compared to the risk when the plant is shutdown.

<u>Response:</u> There is no increase in risk due to the fact that the disassembly and inspection of the valve(s) will be performed during a scheduled maintenance outage of the standby auxiliary feedwater system, when the system is already out of service.

Ginna has a total of five sources of safety related auxiliary feedwater (as compared to three in most plants) including: two motor driven auxiliary feedwater pumps, one turbine driven auxiliary feedwater pump, and two standby auxiliary feedwater pumps. Only one of the standby auxiliary feedwater pump trains will be affected.

The existing Ginna Station Technical Specification for the standby auxiliary feedwater trains, LCO 3.7.5, provides for a 14 day allowed outage time. The check valve disassembly can easily be performed within this time window. Also, the inspection and disassembly activities are required to be performed within consideration of the rules involved per 10 CFR 50.65(a)(4).

<u>RAI 6:</u> Provide sufficient information for NRC staff to reach a safety or risk determination with regards to isolation of these check valves when testing on-line. Also provide copies of drawings (P&IDs) showing check valves and isolation valves and provide details how isolation of these check valves will be established.

<u>Response:</u> As noted in response 1 above, these valves will not be full flow tested. As shown on the attached drawings (33013-1250 sheet 2 of 3 coordinates B-9 and B-10, and 33013-1238 coordinates B-1 and I-1) these valves are in the service water line to the standby auxiliary feedwater pumps. The standby auxiliary feedwater pump will be isolated at the time that the valve inspection and disassembly take place. On the service water side, isolation will be performed by a 4 inch isolation valve (9626A or 9626B as shown on drawing 33013-1250 sheet 2 of 3 coordinates A-2 and B-3). There is no

difference in isolation, regardless of the plant status. On the downstream side there are a number of isolation valves between the check valve and the steam generator (including three check valves and numerous isolation valves as shown on drawing 33013-1238).

<u>RAI 7:</u> Provide the leak testing experience and leak tightness reliability of the associated isolation valves and the potential consequences of a loss of isolation capability during disassembly, inspection, and manual exercising of check valves 9627A and 9627B when the plant is online.

<u>Response:</u> Regardless of whether the valves are inspected and disassembled off-line or on-line they must be isolated from the service water loop. The service water isolation valves have historically performed well as an isolation barrier. The isolation valves between the check valve and the steam generator have also historically performed well as an isolation barrier and are the same isolation valves that are used when performing pump maintenance.

<u>RAI 8:</u> Based on the risk significance discussed in RAI 5 above, discuss what preventive or compensatory measures are necessary to maintain safety and minimize risk while performing on-line IST.

<u>Response:</u> The draining of the lines for the valve inspection and disassembly is accomplished through existing local manual drain and vent valves, such that there is a positive and controlled means of determining the status of the isolation boundary. The isolation valves provide sufficient isolation of the affected SAFW train from in-service systems. No addition preventative or compensatory measures are required as a result of the valve disassembly.

Note: The OM-10 para 4.2.1.1(e) states that "if exercising is not practicable during plant operation or cold shutdowns, it may be limited to full-stroke during refueling outage." Therefore, the inservice testing (IST) of the check valves must coincide with refueling outage of 18 months. If the licensee changes refueling outage to 24 months as allowed by the Technical Specification, the IST of these valves must coincide with the refueling outage of 24 months. This change can be done without any additional relief requests for these valves.

<u>Response:</u> As part of the 1996 conversion of the Ginna Technical Specifications to the format of NUREG-1431, Ginna was evaluated and approved for nominal 24 month surveillance intervals. Though the current nominal cycle length is 18 months, the IST program allows for a refueling outage cycle length test frequency of up to 24 months. The attached Relief Requests VR-1 and VR-2 have been revised to remove the reference to cycle length.

R.E. Ginna Station, Fourth Interval Inservice Testing Program

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RELIEF REQUEST NO. VR - 1____

SYSTEM:	Emergency Diesel Generator Fuel Oil	
VALVES:	5960A,	, 5960B
CATEGORY:	С	
SAFETY CLASS:	3	
FUNCTION:	These the fue close to recircu	check valves open to provide a flow path for overflow from I oil day tank to the fuel oil storage tank. These valves o prevent reverse flow into the fuel oil day tank during lation of the fuel oil storage tank.
TEST REQUIREMENT:	American Society of Mechanical Engineers (ASME) Code for Operations and Maintenance of Nuclear Power Plants (OM Code), 1987 Edition (ASME/ANSI-1987), Part 10, "Inservice Testing of Valves in Light-Water Reactor Power Plants" (OM-10), paragraph 4.3.2.4(c) states "As an alternative to the testing in (a) and (b) above, disassembly every refueling outage to verify operability of check valves may be used."	
BASIS FOR RELIEF:	Relief is requested to disassemble, manually full stroke exercise and inspect one check valve on a rotating basis, at a frequency of each operating cycle in lieu of during each refueling outage. This is to allow the surveillance requirement to be met with the plant online. The following underscore the usefulness and applicability of an online testing approach:	
	1.	The design of the system is such that either emergency diesel generator can be isolated and the check valve disassembled with the plant online.
	2.	Performing the inspection with the plant online reduces outage complexity.
	3.	The check valves are located in an area where performance of the disassembly coupled with other major outage work increases the potential development of error- likely situations in work control and reassembly processes.
	4.	The check valve disassembly and inspection activities can be completed within 50% or less of the associated system Technical Specification allowed outage time.

- 5. An acceptable testing frequency can be maintained separately without being tied directly to a refueling outage. Inservice testing on a frequency that maintains the acceptable time period between testing activities during the operating cycle is consistent with the intent of OM-10 and GL 89-04.
- 6. The number of tests to be performed using either the outage or online frequency statements should be approximately equivalent. Thus, an equivalent level of quality and safety is maintained.
- ALTERNATE TESTING: One valve will be disassembled, manually full-stroke exercised and inspected once each operating cycle on a rotating basis. If that valve fails, the remaining valve will be disassembled, fullstroke exercised and inspected for operability at that same time. (re: Generic Letter 89-04, Attachment 1 - Position 2)

R.E. Ginna Station, Fourth Interval Inservice Testing Program

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RELIEF REQUEST NO. VR - 2

SYSTEM:	Standby Auxiliary Feedwater		
VALVES:	9627A, 9627B		
CATEGORY:	С		
SAFETY CLASS:	3		
FUNCTION:	These s flow from back int flow pat	service water suction check valves close to prevent reverse m Standby Auxiliary Feedwater (SAFW) System piping to the Service Water (SW) System and open to provide a th for service water to the SAFW pumps.	
TEST REQUIREMENT:	American Society of Mechanical Engineers (ASME) Code for Operations and Maintenance of Nuclear Power Plants (OM Code), 1987 Edition (ASME/ANSI-1987), Part 10, "Inservice Testing of Valves in Light-Water Reactor Power Plants" (OM-10), paragraph 4.3.2.4(c) states "As an alternative to the testing in (a) and (b) above, disassembly every refueling outage to verify operability of check valves may be used."		
BASIS FOR RELIEF:	Relief is requested to disassemble, manually full stroke exercise and inspect one check valve on a rotating basis, at a frequency of each operating cycle in lieu of during each refueling outage. This is to allow the surveillance requirement to be met with the plant online. The following underscore the usefulness and applicability of an online testing approach:		
	1.	The design of the system is such that either Standby Auxiliary Feedwater pump can be isolated and the check valve disassembled with the plant online.	
	2.	Performing the inspection with the plant online reduces outage complexity.	
	3.	The check valves are located in an area where performance of the disassembly coupled with other major outage work increases the potential development of error- likely situations in work control and reassembly processes.	
	4 .	The check valve disassembly and inspection activities can be completed within 50% or less of the associated system Technical Specification allowed outage time.	

- 5. An acceptable testing frequency can be maintained separately without being tied directly to a refueling outage. Inservice testing on a frequency that maintains the acceptable time period between testing activities during the operating cycle is consistent with the intent of OM-10 and GL 89-04.
- 6. The number of tests to be performed using either the outage or online frequency statements should be approximately equivalent. Thus, an equivalent level of quality and safety is maintained.
- ALTERNATE TESTING: Partial stroke exercising will be performed each quarter. One valve will be disassembled, manually full-stroke exercised and inspected each operating cycle on a rotating basis. If that valve fails, the remaining valve will be disassembled, full-stroke exercised and inspected for operability at that same time. (re. Generic Letter 89-04 Position 2).







