

July 15, 2004

Mr. Michael R. Kansler, President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING USE OF LATER EDITION OF THE AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME) CODE REQUIREMENTS FOR EXAMINATION OF REACTOR VESSEL CLOSURE STUDS, JAMES A. FITZPATRICK NUCLEAR POWER PLANT, INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3, AND PILGRIM NUCLEAR POWER STATION (TAC NOS. MC2637, MC2638, MC2639, AND MC2640)

Dear Mr. Kansler:

By letter dated April 14, 2004, Entergy Nuclear Operations, Inc. (Entergy) submitted relief request (RR) Nos. 33, 71, 3-40(A), and 41 for the James A. FitzPatrick Nuclear Power Plant, Indian Point Nuclear Generating Unit Nos. 2 and 3, and Pilgrim Nuclear Power Station, respectively. In these RRs, Entergy proposed to use the requirements in the 1998 Edition, 2000 Addenda, of the ASME Code, Section XI, Table IWB-2500-1, Category B-G-1, Item B6.30 for the examination of the reactor vessel closure studs, when removed.

The Nuclear Regulatory Commission staff has been reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). During a telephone call on July 12, 2004, the Entergy staff indicated that a response to the RAI would be provided within 45 days.

Sincerely,

/RA/

Patrick D. Milano, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-333, 50-247, 50-286, and 50-293

Enclosure: As stated

cc w/encl: See next page

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Mr. Michael R. Kansler, President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
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Dear Mr. Kansler:

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Docket Nos. 50-333, 50-247, 50-286, and 50-293

Enclosure: As stated

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REQUEST FOR ADDITIONAL INFORMATION (RAI)

REGARDING RELIEF REQUEST NOS. 33, 71, 3-40, AND 41

ENTERGY NUCLEAR OPERATIONS, INC.

JAMES A. FITZPATRICK NUCLEAR POWER PLANT, INDIAN POINT NUCLEAR

GENERATING UNIT NOS. 2 AND 3, AND PILGRIM NUCLEAR POWER STATION

DOCKET NOS. 50-333, 50-247, 50-286, AND 50-293

By letter dated April 14, 2004, Entergy Nuclear Operations, Inc. requested approval to use the requirements in the 1998 Edition, 2000 Addenda of the ASME Code, Section XI, Table IWB-2500-1, Category B-G-1, Item B6.30 for the examination of the reactor vessel closure studs, when removed. The Nuclear Regulatory Commission (NRC) staff has the following questions regarding the information provided:

1. In accordance with the NRC staff guidance in Office of Nuclear Reactor Regulation Office Instruction LIC-102, "Relief Request Reviews," provide the following for each relief request:
 - a. the requirements (method of examination) of the currently applicable Code Edition and Addenda for each facility.
 - b. the requirements (method of examination) of the proposed Code Edition and Addenda for each facility.
2. Provide the technical basis for eliminating one or the other of the examinations required by the currently applicable Code Edition and Addenda for each facility.
3. Is there a change in the required examination volume or examination surface area between the currently applicable Code Edition and Addenda for each facility and the proposed Code Edition and Addenda?

Enclosure

FitzPatrick Nuclear Power Plant

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