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Dresden Nuclear Power Station
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10 CFR 50.73

June 28, 2004

SVPLTR: #04-0041

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

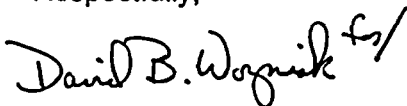
Dresden Nuclear Power Station, Unit 2
Facility Operating License No. DRP-19
NRC Docket No. 50-237

Subject: Licensee Event Report 2004-004-00, "Unit 2 Manual SCRAM Due To The Trip Of A Reactor Recirculation Pump"

Enclosed is Licensee Event Report 2004-004-00, "Unit 2 Manual SCRAM Due To The Trip Of A Reactor Recirculation Pump," for Dresden Nuclear Power Station. This event is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A), "Any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph (a)(2)(iv)(B) of this section."

Should you have any questions concerning this report, please contact Jeff Hansen, Regulatory Assurance Manager, at (815) 416-2800.

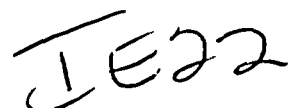
Respectfully,



Danny G. Bost
Site Vice President
Dresden Nuclear Power Station

Enclosure

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Dresden Nuclear Power Station



NRC FORM 366 (7-2001)			U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OBM NO. 3150-0104 EXP 7-31-2004								
LICENSEE EVENT REPORT (LER)									Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to bjs1@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.					
1. FACILITY NAME Dresden Nuclear Power Station Unit 2						2. DOCKET NUMBER 05000237			3. PAGE 1 of 3					
4. TITLE Unit 2 Manual Scram Due To The Trip Of A Reactor Recirculation Pump														
5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED					
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER				
04	28	2004	2004	- 004 -	00	06	28	2004	N/A	N/A				
9. OPERATING MODE		11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)												
1		20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)						
10. POWER LEVEL		20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)		50.73(a)(2)(x)						
066		20.2203(a)(1)		50.36(c)(1)(i)(A)		X 50.73(a)(2)(iv)(A)		73.71(a)(4)						
		20.2203(a)(2)(I)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)		73.71(a)(5)						
		20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)		OTHER						
		20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)		Specify in Abstract below or in NRC Form 366A						
		20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)								
		20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)								
		20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)								
		20.2203(a)(3)(I)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)								
12. LICENSEE CONTACT FOR THIS LER														
NAME George Papanic Jr.						TELEPHONE NUMBER (Include Area Code) (815) 416-2815								
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT														
CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX					
14. SUPPLEMENTAL REPORT EXPECTED														
X	YES (If yes, complete EXPECTED SUBMISSION DATE)				NO	15. EXPECTED SUBMISSION DATE		MONTH	DAY	YEAR				
								08	27	2004				

16. ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On April 28, 2004, at 1536 hours (CDT), with Unit 2 at approximately 66 percent power in Mode 1, a manual scram was inserted by operations personnel due to the plant being in a region of the reactor's power to flow map that required an immediate scram. The power to flow condition was caused by the unexpected trip of the 2A Reactor Recirculation Pump and the manual scram was in accordance with immediate operator actions specified in procedure DOA 202.01, "Recirculation (RECIRC) Pump Trip - One Or Both Pumps." There were no Electromatic or Safety Relief valve actuations and no Emergency Core Cooling System initiations. Primary Containment Isolation System Group 2 and 3 isolations occurred as expected due to normal reactor water level decrease following the scram. All control rods fully inserted and all other systems responded to the manual scram as expected.

The investigation of the root cause and the associated corrective actions to prevent reoccurrence of the event is currently ongoing. The results will be described in a supplemental report scheduled to be submitted no later than August 27, 2004.

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET NUMBER	6. LER NUMBER			3. PAGE
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Dresden Nuclear Power Station Unit 2	05000237	2004	004	00	2 of 3

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

Dresden Nuclear Power Station Unit 2 is a General Electric Company Boiling Water Reactor with a licensed maximum power level of 2957 megawatts thermal. The Energy Industry Identification System codes used in the text are identified as [XX].

A. Plant Conditions Prior to Event:

Unit: 02	Event Date: 4-28-2004	Event Time: 1536 CDT
Reactor Mode: 1	Mode Name: Power Operation	Power Level: 66 percent
Reactor Coolant System Pressure: 1000 psig		

B. Description of Event:

On April 28, 2004, at 1536 hours (CDT), with Unit 2 at approximately 66 percent power in Mode 1, a manual scram was inserted by operations personnel due to the plant being in a region of the reactor's power to flow map that required an immediate scram. The power to flow condition was caused by the unexpected trip of the 2A Reactor Recirculation Pump [P] and the manual scram was in accordance with immediate operator actions specified in procedure DOA 202.01, "Recirculation (RECIRC) Pump Trip – One Or Both Pumps." There were no Electromatic or Safety Relief valve actuations and no Emergency Core Cooling System initiations. Primary Containment Isolation System Group 2 and 3 isolations occurred as expected due to normal reactor water level decrease following the scram. All control rods fully inserted and all other systems responded to the manual scram as expected.

An ENS call was made on April 28, 2004, at 1750 hours (CDT) for the above-described event. The assigned ENS event number was 40713.

This event is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A), "Any event or condition that resulted in manual or automatic actuation of any of the systems listed in paragraph (a)(2)(iv)(B) of this section." The manual actuation of the reactor protection system is listed in 10 CFR 50.73(a)(2)(iv)(B).

C. Cause of Event:

The investigation of the root cause and the associated corrective actions to prevent reoccurrence of the event is currently ongoing. Dresden shipped the failed Recirculation Pump motor to a vendor's maintenance shop for evaluation on May 17, 2004. The preliminary results of the investigation indicate that the motor failure was the result of arcing between electrical phases within the motor. The final results of the investigation and corrective actions to prevent reoccurrence will be described in a supplemental report scheduled to be submitted no later than August 27, 2004.

D. Safety Analysis:

The safety significance of the Reactor Recirculation pump trip and the resulting manual scram event was minimal. The transient response of the plant has been analyzed for the trip of one or two Reactor Recirculation pump and the results are contained in Chapter 15 of the Dresden Update Final Safety Analysis Report. The manual scram of the plant was in accordance with an approved procedure. There were no Electromatic or Safety Relief valve actuations and no Emergency Core Cooling System initiations. Primary Containment Isolation System Group 2 and 3 isolations occurred as expected due to normal reactor water level decrease following the scram. All control rods fully inserted and all other systems responded to the manual scram as expected.

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Dresden Nuclear Power Station Unit 2	05000237	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

Therefore, the consequences of this event had minimal impact on the health and safety of the public and reactor safety.

E. Corrective Actions:

The failed Reactor Recirculation pump motor was replaced.

The failed Reactor Recirculation pump motor was shipped on May 17, 2004, to a vendor's maintenance shop for evaluation.

The final corrective actions to prevent reoccurrence will be described in a supplemental report scheduled to be submitted no later than August 27, 2004.

F. Previous Occurrences:

A review of Dresden Nuclear Power Station Licensee Event Reports (LERs) and operating experience over the previous five years did not find any similar occurrences associated with a failed Reactor Recirculation pump.

G. Component Failure Data:

To be provided in a supplemental report.