

Power Reactor

Event # 36404

Site: BRUNSWICK				Notification Date / Time: 11/05/1999 13:28 (EST)		
Unit: 1		Region: 2		State : NC		Event Date / Time: 11/05/1999 10:50 (EST)
Reactor Type: [1] GE-4,[2] GE-4				Last Modification: 11/05/1999		
Containment Type: MARK I MARK I						
NRC Notified by: MICHAEL WILLIAMS				Notifications: PIERCE SKINNER R2		
HQ Ops Officer: STEVE SANDIN						
Emergency Class: NON EMERGENCY						
10 CFR Section:						
50.72(b)(2)(ii)		RPS ACTUATION				
50.72(b)(2)(ii)		ESF ACTUATION				
Unit	Scram Code	RX Crit	Init Power	Initial RX Mode	Curr Power	Current RX Mode
1	M/R	Yes	100	Power Operation	0	Hot Shutdown

Unit 1 was manually scrammed following the unplanned trip of the "1B" Reactor Feed Pump during monthly surveillance testing.

"On November 5, 1999 at 1050 [hours] a manual Reactor Scram was inserted on Unit 1 after Reactor Feed Pump '1B' tripped during feed pump testing and Reactor Water Level was not maintained by the '1A' Reactor Feed Pump. Primary Containment Groups 2, 6 and 8 Isolations were received following the manual reactor scram from low Reactor Water Level. There is normally a level transient following a reactor scram, which is anticipated by the operating crew. Reactor Water Level lowered to 112 inches. The High Pressure Coolant Injection system was manually started, but was not used for injection. All required Isolations occurred as a result of the Reactor Water Level Low Level One initiation signal. Group 2 isolation valves include Drywell Equipment and Floor Drain, Traversing Incore Probe, Residual Heat Removal (RHR) Discharge Isolation to Radwaste and RHR Process Sampling Valves. Group 6 isolation valves include Containment Atmosphere Control System and Post Accident Valves. Group 8 isolation valves include RHR System Shutdown Cooling Isolation Valves, these valves were closed prior to the isolation sign.

"[The initial safety significance is] minimal, all systems functioned as designed. The licensee took conservative action to insert a manual Reactor Scram.

"The cause of the '1B' Reactor Feed Pump trip will be determined and corrected."

All rods fully inserted. Decay heat is being removed via the Main Condenser. The automatic reactor scram level setpoint is 162 inches with HPCI injection occurring at 108 inches.

The licensee informed the NRC Resident Inspector.

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