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U.S. Nuclear Regulatory Commission



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Event Notification Report for February 10, 2000

U.S. Nuclear Regulatory Commission
Operations Center

Event Reports For
02/09/2000 - 02/10/2000

**** EVENT NUMBERS ****

36671 36672 36673 36674 36675 36676 36677

Power Reactor	Event Number: 36671
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FACILITY: QUAD CITIES	REGION: 3	NOTIFICATION DATE: 02/09/2000
UNIT: [] [2] []	STATE: IL	NOTIFICATION TIME: 06:45[EST]
RXTYPE: [1] GE-3, [2] GE-3		EVENT DATE: 02/09/2000
		EVENT TIME: 04:21[CST]
NRC NOTIFIED BY: MACLENNAN		LAST UPDATE DATE: 02/09/2000
HQ OPS OFFICER: CHAUNCEY GOULD		

PERSON	ORGANIZATION
EMERGENCY CLASS: N/A	MICHAEL PARKER R3
10 CFR SECTION: AESF 50.72(b)(2)(ii) ESF ACTUATION	

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
2	N	N	0	Cold Shutdown	0	Cold Shutdown

EVENT TEXT

INVALID ESF ACTUATION CAUSED BY BACKFILLING OF REACTOR LEVEL INSTRUMENTATION

On 02/09/2000 @ 0421 hours, Unit Two received a full reactor SCRAM and a full Group II and Group III isolation, including a Shutdown Cooling isolation. This event occurred as a result of scheduled backfilling of reactor level instrumentation, and is being reported as an invalid ESF actuation.

At the time of the event, all control rods were fully inserted, so no rod motion occurred. All systems responded as designed. Shutdown Cooling was restored at 0432 hours with minimal temperature increase(5pF).

The NRC Resident Inspector was notified.

Power Reactor	Event Number: 36672
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FACILITY: GINNA	REGION: 1	NOTIFICATION DATE: 02/09/2000
UNIT: [1] [] []	STATE: NY	NOTIFICATION TIME: 11:16[EST]
RXTYPE: [1] W-2-LP		EVENT DATE: 02/09/2000

B-6

NRC NOTIFIED BY: GOMEZ		EVENT TIME: 09:15[EST]	
HQ OPS OFFICER: CHAUNCEY GOULD		LAST UPDATE DATE: 02/09/2000	
EMERGENCY CLASS: N/A		PERSON	ORGANIZATION
10 CFR SECTION: APRE 50.72(b)(2)(vi) OFFSITE NOTIFICATION		HAROLD GRAY	R1

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
1	N	Y	100	Power Operation	100	Power Operation

EVENT TEXT

THE LICENSEE NOTIFIED THE NEW YORK DEPARTMENT OF ENVIRONMENTAL CONSERVATION THEY EXCEEDED THEIR DELTA T DISCHARGE LIMIT.

On 2/9/00 @ 0915, the following information was reported as a State Pollutant Discharge Elimination System(SPDES) exception to the New York State DEC per the 24 hour notification requirement. On 2/8/00 from -0630 - 0715 hours, the delta t for the main condenser at the site was above the limit of 28pF, with a maximum of 30.4pF, and estimated that the maximum discharge temperature would have been ~ 63pF. For the period - 0800 - 1100 hours, delta t was again above the 28pF limit, attaining a maximum delta t of 62pF, with an estimated discharge temperature of ~ 95pF. Observations indicate that no significant fish kill occurred.

The NRC Resident Inspector was notified.

Power Reactor		Event Number: 36673	
FACILITY: COMANCHE PEAK		REGION: 4	NOTIFICATION DATE: 02/09/2000
UNIT: [1] [2] []		STATE: TX	NOTIFICATION TIME: 12:35[EST]
RXTYPE: [1] W-4-LP, [2] W-4-LP		EVENT DATE: 02/09/2000	
NRC NOTIFIED BY: TOM BEAUDIN		EVENT TIME: 11:33[CST]	
HQ OPS OFFICER: STEVE SANDIN		LAST UPDATE DATE: 02/09/2000	
EMERGENCY CLASS: N/A		PERSON	ORGANIZATION
10 CFR SECTION: AARC 50.72(b)(1)(v) OTHER ASMT/COMM INOP		LINDA HOWELL	R4

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
1	N	Y	100	Power Operation	100	Power Operation
2	N	Y	100	Power Operation	100	Power Operation

EVENT TEXT

THE SERVER TO THE EMERGENCY OFFSITE FACILITY FAILED CAUSING THE SAFETY PARAMETER DISPLAY SYSTEM (SPDS) AND EMERGENCY RESPONSE DATA SYSTEM (ERDS) TO BECOME INOPERABLE

At 1645 CT on February 8, 2000 the server which transfers Plant Computer data to the two SDSs in the EOF and the ERDS modems failed. This caused the Safety Parameter Display System (SPDS) and the Emergency Response Data System (ERDS) to become unavailable in the EOF as specified in the FSAR section III.A.1.2. The server was replaced by Telecommunications and SPDS and ERDS restored to the EOF at 0834 CT on February 9, 2000. The total time that SPDS and ERDS were unavailable to the EOF is 15 hours and 49 minutes. The failure of the server occurred at the same time as a report of a fire at the Nuclear Operations Support Facility (the EOF is located inside)

building.

The server was replaced by Telecommunication. The SPDS and ERDS are available at the EOF.

The NRC Resident Inspector was notified of this event by the Licensee.

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Power Reactor | Event Number: 36674

FACILITY: SAINT LUCIE	REGION: 2	NOTIFICATION DATE: 02/09/2000
UNIT: [1] [2] [1]	STATE: FL	NOTIFICATION TIME: 13:49[EST]
RXTYPE: [1] CE, [2] CE		EVENT DATE: 02/09/2000
		EVENT TIME: 12:45[EST]
NRC NOTIFIED BY: DAVE WILLIAMS		LAST UPDATE DATE: 02/09/2000
HQ OPS OFFICER: STEVE SANDIN		

EMERGENCY CLASS:	N/A	PERSON	ORGANIZATION
10 CFR SECTION:		MARK LESSER	R2
APRE 50.72(b)(2)(vi)	OFFSITE NOTIFICATION		

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
1	N	Y	100	Power Operation	100	Power Operation
2	N	Y	100	Power Operation	100	Power Operation

EVENT TEXT

OFFSITE NOTIFICATION TO THE FLORIDA DEP INVOLVING THE REMOVAL OF A
LOGGERHEAD SEA TURTLE IN THE PLANT INTAKE CANAL BARRIER NET.

NOTIFICATION MADE TO FLORIDA DEPARTMENT OF ENVIRONMENTAL PROTECTION (DEP)
REGARDING REMOVAL OF LOGGERHEAD TURTLE FROM PLANT INTAKE CANAL BARRIER NET.
THE TURTLE WILL BE SENT OFFSITE FOR REHABILITATION. THERE WERE NO INJURIES
TO THE TURTLE.

THE LICENSEE INFORMED THE FLORIDA DEP AND THE NRC RESIDENT INSPECTOR.

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Power Reactor | Event Number: 36675

FACILITY: LASALLE	REGION: 3	NOTIFICATION DATE: 02/09/2000
UNIT: [] [2] []	STATE: IL	NOTIFICATION TIME: 16:36[EST]
RXTYPE: [1] GE-5, [2] GE-5		EVENT DATE: 02/09/2000
		EVENT TIME: 12:12[CST]
NRC NOTIFIED BY: D. COVEYOU		LAST UPDATE DATE: 02/09/2000
HQ OPS OFFICER: BOB STRANSKY		

EMERGENCY CLASS:	N/A	PERSON	ORGANIZATION
10 CFR SECTION:		MICHAEL PARKER	R3
AIND 50.72(b)(2)(iii)(D)	ACCIDENT MITIGATION		

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
2	N	Y	100	Power Operation	100	Power Operation

EVENT TEXT

EMERGENCY DIESEL GENERATOR TRIPPED FOLLOWING TESTING

*While performing LOS-DG-M3, Monthly Operability Run of 2B Diesel Generator,
Division three, the Diesel Generator tripped on reverse power before the

output breaker was opened while preparing to take the Diesel Generator off line following this surveillance. Concurrently, a local alarm was received indicating a DC power failure. The reverse power trip is a function that is bypassed under accident conditions. Upon investigation, two fuses supplying engine control power were discovered blown. It is not known whether the fuses would have blown with the reverse power bypassed. The fuses which were installed were verified to be the correct type. The fuses were replaced with like for like and the power failure alarm cleared and the installed fuses did not blow. A plan is being developed to investigate the cause of the fuses blowing. The Diesel Generator is not available and is inoperable per LaSalle Technical Specifications, pending troubleshooting and potential corrective actions. The cause of the blown fuses is unknown at this time."

The NRC resident inspector has been informed of this event by the licensee.

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Power Reactor | Event Number: 36676

FACILITY: BEAVER VALLEY REGION: 1 NOTIFICATION DATE: 02/09/2000
 UNIT: [1] [] [] STATE: PA NOTIFICATION TIME: 16:50[EST]
 RXTYPE: [1] W-3-LP, [2] W-3-LP EVENT DATE: 02/08/2000
 NRC NOTIFIED BY: CHRISTOPHER HYNES EVENT TIME: 12:09[EST]
 HQ OPS OFFICER: BOB STRANSKY LAST UPDATE DATE: 02/09/2000

EMERGENCY CLASS:	N/A	PERSON	ORGANIZATION
10 CFR SECTION:		HAROLD GRAY	R1
NINF	INFORMATION ONLY		

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
1	N	Y	93	Power Operation	93	Power Operation

EVENT TEXT

INFORMATION CALL REGARDING RIVER WATER PUMP PROBLEMS

"This report is being made to provide preliminary notification of an event that is currently under investigation at Beaver Valley Power Station Unit No. 1 (BVPS-1).

"River Water Pump WR-P-1B at Beaver Valley Power Station Unit 1 tripped on overcurrent shortly after it was attempted to be started at 1209 on 02/08/2000. The Action Statement for Technical Specification 3/4.7.4.1 was entered for less than two River Water subsystems being operable. River Water Pump WR-P-1C (swing pump) was then aligned to the 'B' train header and it also tripped on overcurrent shortly after it was started at 1716 on 02/08/2000.

"Preliminary information identified that the 1B and 1C River Water Pumps would not operate due to the loss of gap between the pumps rotating assembly and the pump bowl. Investigation into the cause of the loss of the pumps' gap is ongoing.

"BVPS-1 River Water Pump 1A on the 'A' train continues to operate satisfactorily and is operable. The 'A' train is being treated as protected equipment (i.e., no elective work is being done on or near 'A' train components). The BVPS-1 operating crews have reviewed the procedures associated with operating the BVPS-1 Auxiliary River Water System in the Alternate Intake Structure and the use of the cross tie connection with the diesel driven fire pump."

The licensee stated that the unit entered a 72 hour Technical Specification Limiting Condition for Operation (LCO) at 1209 on 2/8/2000. The NRC resident inspector has been informed of this notification by the licensee.

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|Power Reactor | Event Number: 36677 |

FACILITY: SOUTH TEXAS REGION: 4 NOTIFICATION DATE: 02/09/2000
 UNIT: [1] [2] [1] STATE: TX NOTIFICATION TIME: 18:38[EST]
 RXTYPE: [1] W-4-LP, [2] W-4-LP EVENT DATE: 02/09/2000
 NRC NOTIFIED BY: TIM FRAWLEY EVENT TIME: 15:46[CST]
 HQ OPS OFFICER: BOB STRANSKY LAST UPDATE DATE: 02/09/2000

EMERGENCY CLASS:	N/A	PERSON	ORGANIZATION
10 CFR SECTION:		LINDA HOWELL	R4
AESF 50.72(b)(2)(ii)	ESF ACTUATION		

UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MODE	CURR PWR	CURR RX MODE
2	N	Y	32	Power Operation	2	Startup

EVENT TEXT

AUXILIARY FEEDWATER SYSTEM ACTUATION DUE TO UNEXPECTED AMSAC ACTUATION

A main turbine trip and auxiliary feedwater system actuation occurred due to an unexpected ATWS mitigation system actuation circuitry (AMSAC) actuation. Because the reactor was operating below 50% power, the turbine trip did not cause an automatic reactor trip. The licensee is currently investigating the cause of the AMSAC actuation.

The NRC resident inspector has been informed of this notification by the licensee.

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