

TABLE OF CONTENTS

| | | |
|--------|---|----------|
| 3.5.5 | Trisodium Phosphate (TSP) | 3.5.5-1 |
| 3.6 | CONTAINMENT SYSTEMS..... | 3.6.1-1 |
| 3.6.1 | Containment..... | 3.6.1-1 |
| 3.6.2 | Containment Air Locks..... | 3.6.2-1 |
| 3.6.3 | Containment Isolation Valves..... | 3.6.3-1 |
| 3.6.4 | Containment Pressure..... | 3.6.4-1 |
| 3.6.5 | Containment Air Temperature..... | 3.6.5-1 |
| 3.6.6 | Containment Spray and Cooling Systems..... | 3.6.6-1 |
| 3.6.7 | Hydrogen Recombiners..... | 3.6.7-1 |
| 3.6.8 | Iodine Removal System (IRS)..... | 3.6.8-1 |
| 3.7 | PLANT SYSTEMS..... | 3.7.1-1 |
| 3.7.1 | Main Steam Safety Valves (MSSVs)..... | 3.7.1-1 |
| 3.7.2 | Main Steam Isolation Valves (MSIVs)..... | 3.7.2-1 |
| 3.7.3 | Auxiliary Feedwater (AFW) System..... | 3.7.3-1 |
| 3.7.4 | Condensate Storage Tank (CST)..... | 3.7.4-1 |
| 3.7.5 | Component Cooling (CC) System..... | 3.7.5-1 |
| 3.7.6 | Service Water (SRW) System..... | 3.7.6-1 |
| 3.7.7 | Saltwater (SW) System..... | 3.7.7-1 |
| 3.7.8 | Control Room Emergency Ventilation System (CREVS) .. | 3.7.8-1 |
| 3.7.9 | Control Room Emergency Temperature System (CRETS) .. | 3.7.9-1 |
| 3.7.10 | Emergency Core Cooling System (ECCS) Pump Room Exhaust Filtration System (PREFS) | 3.7.10-1 |
| 3.7.11 | Spent Fuel Pool Exhaust Ventilation System (SFPEVS) | 3.7.11-1 |
| 3.7.12 | Penetration Room Exhaust Ventilation System (PREVS) | 3.7.12-1 |
| 3.7.13 | Spent Fuel Pool (SFP) Water Level..... | 3.7.13-1 |
| 3.7.14 | Secondary Specific Activity..... | 3.7.14-1 |
| 3.7.15 | Main Feedwater Isolation Valves (MFIVs)..... | 3.7.15-1 |
| 3.7.16 | Spent Fuel Pool (SFP) Boron Concentration..... | 3.7.16-1 |
| 3.8 | ELECTRICAL POWER SYSTEMS..... | 3.8.1-1 |
| 3.8.1 | AC Sources—Operating | 3.8.1-1 |
| 3.8.2 | AC Sources—Shutdown | 3.8.2-1 |
| 3.8.3 | Diesel Fuel Oil..... | 3.8.3-1 |
| 3.8.4 | DC Sources—Operating | 3.8.4-1 |
| 3.8.5 | DC Sources—Shutdown | 3.8.5-1 |
| 3.8.6 | Battery Cell Parameters | 3.8.6-1 |
| 3.8.7 | Inverters—Operating | 3.8.7-1 |
| 3.8.8 | Inverters—Shutdown | 3.8.8-1 |

TABLE OF CONTENTS

| | | |
|--------|---|----------|
| 3.8.9 | Distribution Systems—Operating | 3.8.9-1 |
| 3.8.10 | Distribution Systems—Shutdown | 3.8.10-1 |
| 3.9 | REFUELING OPERATIONS..... | 3.9.1-1 |
| 3.9.1 | Boron Concentration..... | 3.9.1-1 |
| 3.9.2 | Nuclear Instrumentation..... | 3.9.2-1 |
| 3.9.3 | Containment Penetrations..... | 3.9.3-1 |
| 3.9.4 | Shutdown Cooling (SDC) and Coolant Circulation— High Water Level | 3.9.4-1 |
| 3.9.5 | Shutdown Cooling (SDC) and Coolant Circulation— Low Water Level | 3.9.5-1 |
| 3.9.6 | Refueling Pool Water Level..... | 3.9.6-1 |
| 4.0 | DESIGN FEATURES..... | 4.0-1 |
| 4.1 | Site Location..... | 4.0-1 |
| 4.2 | Reactor Core..... | 4.0-1 |
| 4.3 | Fuel Storage..... | 4.0-2 |
| 5.0 | ADMINISTRATIVE CONTROLS..... | 5.1-1 |
| 5.1 | Responsibility..... | 5.1-1 |
| 5.2 | Organization..... | 5.2-1 |
| 5.2.1 | Onsite and Offsite Organizations..... | 5.2-1 |
| 5.2.2 | Unit Staff..... | 5.2-2 |
| 5.3 | Unit Staff Qualifications..... | 5.3-1 |
| 5.4 | Procedures..... | 5.4-1 |
| 5.5 | Programs and Manuals..... | 5.5-1 |
| 5.5.1 | Offsite Dose Calculation Manual..... | 5.5-1 |
| 5.5.2 | Primary Coolant Sources Outside Containment..... | 5.5-2 |
| 5.5.3 | Post-Accident Sampling..... | 5.5-2 |
| 5.5.4 | Radioactive Effluent Controls Program..... | 5.5-3 |
| 5.5.5 | Component Cyclic or Transient Limit..... | 5.5-6 |
| 5.5.6 | Concrete Containment Tendon Surveillance program... .. | 5.5-6 |
| 5.5.7 | Reactor Coolant Pump Flywheel Inspection Program... .. | 5.5-6 |
| 5.5.8 | Inservice Testing Program..... | 5.5-6 |
| 5.5.9 | Steam Generator Tube Surveillance Program..... | 5.5-7 |
| 5.5.10 | Secondary Water Chemistry Program..... | 5.5-17 |
| 5.5.11 | Ventilation Filter Testing Program..... | 5.5-17 |
| 5.5.12 | Explosive Gas and Storage Tank Radioactivity Monitoring Program | 5.5-20 |
| 5.5.13 | Diesel Fuel Oil Testing Program..... | 5.5-21 |
| 5.5.14 | Technical Specifications Bases Control Program..... | 5.5-21 |
| 5.5.15 | Safety Function Determination Program (SFDP)..... | 5.5-22 |

TABLE OF CONTENTS

| | | |
|--------|---|--------|
| 5.5.16 | Containment Leakage Rate Testing Program..... | 5.5-23 |
| 5.6 | Reporting Requirements..... | 5.6-1 |
| 5.6.1 | Occupational Radiation Exposure Report..... | 5.6-1 |
| 5.6.2 | Annual Radiological Environmental Operating report. | 5.6-1 |
| 5.6.3 | Radioactive Effluent Release Report..... | 5.6-2 |
| 5.6.4 | Monthly Operating Reports..... | 5.6-3 |
| 5.6.5 | CORE OPERATING LIMITS REPORT (COLR)..... | 5.6-3 |
| 5.6.6 | Not Used..... | 5.6-9 |
| 5.6.7 | Post-Accident Monitoring Report..... | 5.6-9 |
| 5.6.8 | Tendon Surveillance Report..... | 5.6-9 |
| 5.6.9 | Steam Generator Tube Inspection Report..... | 5.6-9 |

3.7 PLANT SYSTEMS

3.7.16 Spent Fuel Pool (SFP) Boron Concentration

LCO 3.7.16 Boron concentration of the SFP shall be \geq 2000 ppm.

APPLICABILITY: When fuel assemblies are stored in the SFP.

ACTIONS

| CONDITION | REQUIRED ACTION | COMPLETION TIME |
|--|---|-----------------|
| A. Spent Fuel Pool boron concentration not within limit. | ----- NOTE----- LCO 3.0.3 is not applicable. ----- | |
| | A.1 Suspend movement of fuel assemblies in the SFP. | Immediately |
| | <u>AND</u> A.2 Initiate action to restore boron concentration to within limit. | Immediately |

SURVEILLANCE REQUIREMENTS

| SURVEILLANCE | FREQUENCY |
|--|-----------|
| SR 3.7.16.1 Verify boron concentration is greater than 2000 ppm. | 7 days |

4.0 DESIGN FEATURES

4.2.2 Control Element Assemblies

The reactor core shall contain 77 control element assemblies.

4.3 Fuel Storage

4.3.1 Criticality

4.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

- a. Fuel assemblies having a maximum U-235 enrichment of 5.00 weight percent for the Unit 1 pool and 4.52 weight percent for the Unit 2 pool;
- b. For Unit 1, $k_{eff} < 1.00$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.7.2 of the Updated Final Safety Analysis Report (UFSAR) and $k_{eff} \leq 0.95$ if fully flooded with water borated to 350 ppm, which includes an allowance for uncertainties as described in Section 9.7.2 of the UFSAR;
- c. For Unit 2, $k_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.7.2 of the UFSAR;
- d. A nominal 10-3/32-inch center-to-center distance between fuel assemblies placed in the high density fuel storage racks;

4.3.1.2 The new fuel storage racks are designed and shall be maintained with:

- a. Fuel assemblies having a maximum U-235 enrichment of 5.0 weight percent;

4.0 DESIGN FEATURES

- b. $k_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.7.1 of the UFSAR;
- c. $k_{eff} \leq 0.95$ if moderated by aqueous foam, which includes an allowance for uncertainties as described in Section 9.7.1. of the UFSAR; and
- d. A nominal 18-inch center-to-center distance between fuel assemblies placed in the storage racks.

4.3.2 Drainage

The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 63 ft.

4.3.3 Capacity

The spent fuel storage pool is designed and shall be maintained with a storage capacity, for both Units 1 and 2, limited to no more than 1830 fuel assemblies.

| <u>Amendment No.</u> | <u>Additional Conditions</u> | <u>Implementation Date</u> |
|----------------------|--|---|
| 246 | This amendment requires the licensee to incorporate in the Updated Final Safety Analysis Report (UFSAR) changes associated with the aircraft hazards analysis which was evaluated by the staff in the Safety Evaluation dated August 29, 2001. | Next update of the UFSAR |
| 248 | <p>This amendment requires the licensee to incorporate in the Updated Final Safety Analysis Report (UFSAR) changes associated with the loss of feedwater flow analysis which was evaluated by the staff in the safety evaluation dated February 26, 2002.</p> <p>This amendment requires the licensee develop a long-term coupon surveillance program for the Carborundum samples. This program must verify that the Carborundum degradation rates assumed in the licensee's analyses to prove subcriticality, as required by 10 CFR 50.68, remain valid over the seventy-year life span of the Unit 1 spent fuel pool. The licensee must submit this modified coupon surveillance program to the NRC under the 10 CFR 50.90 requirements for its review and approval.</p> | <p>Next update of the UFSAR</p> <p>3 years after approval of this amendment</p> |