



MAY 27 2004

Serial: HNP-04-083

U.S. Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT NO. 1
DOCKET NO. 50-400/LICENSE NO. NPF-63
15-DAY SPECIAL REPORT
STEAM GENERATOR TUBE PLUGGING

Ladies and Gentlemen:

In accordance with Technical Specification 4.4.5.5.a of the Harris Nuclear Plant (HNP), Carolina Power and Light Company doing business as Progress Energy Carolinas, Inc., provides the attached 15-day special report, which reports the number of tubes plugged in each steam generator (SG) following the completion of a SG tube inservice inspection conducted during a mid-cycle outage in May 2004. In addition, this letter provides a correction to the one-year special report of steam generator tube inservice inspection results for refueling outage (RFO) 11 (May 2003) reported by letter HNP-04-076 dated May 7, 2004.

The complete results of the SG tube inservice inspection conducted during the mid-cycle outage in May 2004 will be submitted to the NRC within 12 months of completion as required by Technical Specification 4.4.5.5.b.

Attachment 1 provides the 15-day special report identifying the number of tubes plugged in each steam generator.

Attachment 2 provides the correction to the one-year special report of steam generator tube inservice inspection results for RFO-11 (May 2003).

Please refer any question regarding this submittal to Mr. John Caves at (919) 362-3137.

Sincerely,

A handwritten signature in black ink that reads 'Brian C. McCabe'.

B. C. McCabe
Supervisor – Licensing/Regulatory Programs (Acting)
Harris Nuclear Plant

BCM/jpy

Progress Energy Carolinas, Inc.
Harris Nuclear Plant
P.O. Box 165
New Hill, NC 27562

1047

HNP-04-083

Page 2

Attachments:

- 1. 15-Day Special Report Identifying the Number of Tubes Plugged in Each Steam Generator**
- 2. Correction to the one-year special report of steam generator tube inservice inspection results for RFO-11 (May 2003)**

c:

Mr. R. A. Musser, NRC Senior Resident Inspector

Mr. C. P. Patel, NRC Project Manager

Mr. L. A. Reyes, NRC Regional Administrator

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT NO. 1
 DOCKET NO. 50-400/LICENSE NO. NPF-63
 15-DAY SPECIAL REPORT
 NUMBER OF TUBES PLUGGED IN EACH STEAM GENERATOR

Technical Specification 4.4.5.5.a of the Harris Nuclear Plant (HNP) requires that within 15 days following the completion of each inservice inspection of steam generator (SG) tubes, the number of tubes plugged in each steam generator shall be reported to the Commission in a Special Report pursuant to Specification 6.9.2 (10 CFR 50.4).

The first inservice inspection of the SGs following replacement in refueling outage (RFO) Number 10 (Fall 2001) was conducted during RFO-11 and completed on May 7, 2003. The second inservice inspection of the SGs was conducted during a mid-cycle outage and completed on May 14, 2004.

Number of Tubes Plugged

<u>Steam Generator</u>	<u>A</u>	<u>B</u>	<u>C</u>
Current (May 2004 only)	0	0	3
Cummulative (through May 2004)	1	1	3

Identification of tubes plugged:

SG "A":

One tube plugged (pre-service): R106C85

SG "B":

One tube plugged (pre-service): R114C73

SG "C":

Three tubes plugged in May 2004: R01C120, R02C121, and R03C120

**SHEARON HARRIS NUCLEAR POWER PLANT, UNIT NO. 1
DOCKET NO. 50-400/LICENSE NO. NPF-63
CORRECTION TO THE ONE-YEAR SPECIAL REPORT
STEAM GENERATOR TUBE INSERVICE INSPECTION RESULTS FOR RFO-11 (MAY 2003)**

Technical Specification 4.4.5.5.b of the Harris Nuclear Plant (HNP) requires the complete results of the steam generator (SG) tube inservice inspection shall be submitted in a special report within 12 months following completion of the inspection. The special report shall include:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection, and
3. Identification of tubes plugged.

On May 7, 2004, HNP provided the required special report by letter HNP-04-076. Upon further review of the eddy current testing (ECT) data for refueling outage (RFO) number 11, it was discovered that one indication was missed on SG "C." Based upon review of the RFO-11 ECT (bobbin probe) data, the first sentence of Item 2 on Attachment 1, Page A1-2 of 2, of the special report should be deleted and replaced with the following:

2. Location and percent of wall-thickness penetration for each indication of an imperfection:

~~No recordable eddy current testing (ECT) indications with wall thickness penetration were identified from the results of this first inservice inspection. [Original Text]~~

One recordable eddy current testing (ECT) indication with approximately 37% wall-thickness penetration was identified from the results of this first inservice inspection. [Replaced Text]

Row 1 Column 120 indicated an NQI (non-quantifiable indication) located 0.64 inches above the Cold Leg top-of-tubesheet. Since this indication was missed at the time of the inspections, rotating coil examinations were not used to characterize the actual size and depth of this indication. Based on only the bobbin probe data that is available for this indication, the best estimate of wall-thickness penetration is approximately 37% with the primary frequency, 630 kHz. This depth estimate is derived from comparison to the American Society of Mechanical Engineers (ASME) calibration standard, and is not a qualified depth sizing technique. The missed indication data is as follows:

ROW	COL	VOLTS	DEG	IND	CHN	LOCN	INCH1	BEGT	ENDT	PDIA	PTYPE	CAL	LEG
1	120	0.43	126	NQI	1	TSC	+0.64	09H	TEC	.560	ZBACS	4	C

100% of the SG tubes were examined during RFO-11, and a re-review of the data did not identify any other missed indications that require reporting.