

**MAY 18 2004**

LR-N04-0214



United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

**REQUEST FOR ADDITIONAL INFORMATION  
INSERVICE INSPECTION PROGRAM  
RELIEF REQUEST HC-RR-I2-023  
HOPE CREEK GENERATING STATION  
FACILITY OPERATING LICENSES NPF-57  
DOCKET NOS. 50-354**

Reference: Letter LR-N04-0137, Inservice Inspection Program Relief Request HC-RR-I2-023 Hope Creek Generating Station, dated March 23, 2004

On March 30, 2004 the NRC verbally granted approval of Relief Request HC-RR-I2-023, Section XI Leakage Test. Subsequent to that approval, the NRC requested additional clarifying information regarding, 1) performance of the System Leakage Test with or without insulation and 2) bases for the use of ASME Code Case N-389-1.

In letter (LR-N04-0137) we indicated that a VT-2 examination on the mechanical joints of the SRV assemblies and CRD flanges would be performed during the normal operational start-up sequence at a minimum of 900 psig following a 10 minute hold time (for uninsulated components) in lieu of the nominal operating pressure associated with 100% reactor power (approximately 1005 psig). This one time only test was performed with the insulation removed and then reinstalled after completion of the test.

The first ten-year Inservice Inspection Interval for the Salem Generating Station was based on The 1974 Edition of Section XI including the Summer 1975 Addenda. In a letter dated July 21, 1988 PSEG informed the NRC that we would be using code case N-389 to permit repairs and replacements to be made to a later edition of the ASME Code (1983 Edition including Summer 1983 Addenda). The NRC issued a Safety Evaluation (SE) on May 16, 1989 for Hope Creek Generating Station and Salem Generating Station Units 1&2 for Inservice Inspection Program Relief Requests. The NRC indicated that the NRC has approved Code Case N-389, "Alternate Rules for Repairs, Replacements, or Modifications". This provides for the use of approved ASME, Section XI, Editions and Addenda, subsequent to that specified in the Owners Inservice Inspection Program. Code Case N-389 was incorporated into a later Edition of ASME Section XI.

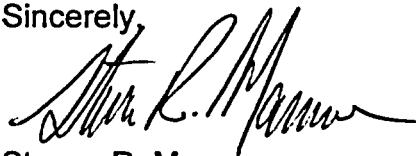
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The second ten-year Inservice Inspection interval for Hope Creek Generating Station is based upon the 1989 Edition of Section XI of the ASME Code. In Revision 13 of Regulatory Guide 1.147 the NRC approved Code Case N-389-1, "Alternate Rules for Repairs, Replacements, or Modifications". SRV assemblies are considered a repair-replacement activity under ASME Section XI, 1995 Edition up to and including the 1996 Addenda. Following repair-replacement a system leakage test is required. Therefore, PSEG will be applying Code Case N-389-1 for use of later Edition and Addenda of ASME Section XI for repair-replacement.

If you have any questions, please contact Mr. Michael Mosier at 856-339-5434.

Sincerely,

A handwritten signature in black ink, appearing to read "Steven R. Mannon". The signature is fluid and cursive, with a long horizontal stroke at the end.

Steven R. Mannon  
Manager – Nuclear Safety and Licensing

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